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VOLUME 4

SYNTHESIS OF CALCULATIONAL METHODS
FOR THE DESIGN AND ANALYSIS OF RADIATION
SHIELDS FOR NUCLEAR ROCKET SYSTEMS

KAP-V

THE POINT KERNEL ATTENUATION PROGRAM

by

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ABSTRACT

This report is Volume 4 of nine volumes of the final report on "Synthesis of Calculational Methods for the Design and Analysis of Radiation Shields for Nuclear Rocket Systems". Presented in this volume is a description of the neutron and gamma ray point kernel attenuation program (KAP-V).

KAP-V is a computer program written in FORTRAN IV language. The program employs the point kernel method to calculate radiation levels at detector points located within or outside a complex radiation source geometry describable by a combination of quadratic surfaces. This program can be used, for example, to calculate gamma ray and/or fast neutron flux, dose, or heating rate. The attenuation function, or kernel, for gamma rays employs exponential attenuation along with a build-up factor. Three optional fast neutron attenuation functions are included: (1) a modified Albert-Welton function for calculating fast neutron dose rate using removal cross sections; (2) a bivariant polynomial expression for computing neutron spectra using infinite media moments data; and (3) a monovariant polynomial expression for computing neutron spectra using infinite media moments data. The program also handles either cylindrical, spherical, disc, line, or point sources. A variety of options are available for describing neutron or gamma ray source distributions in complex geometries.

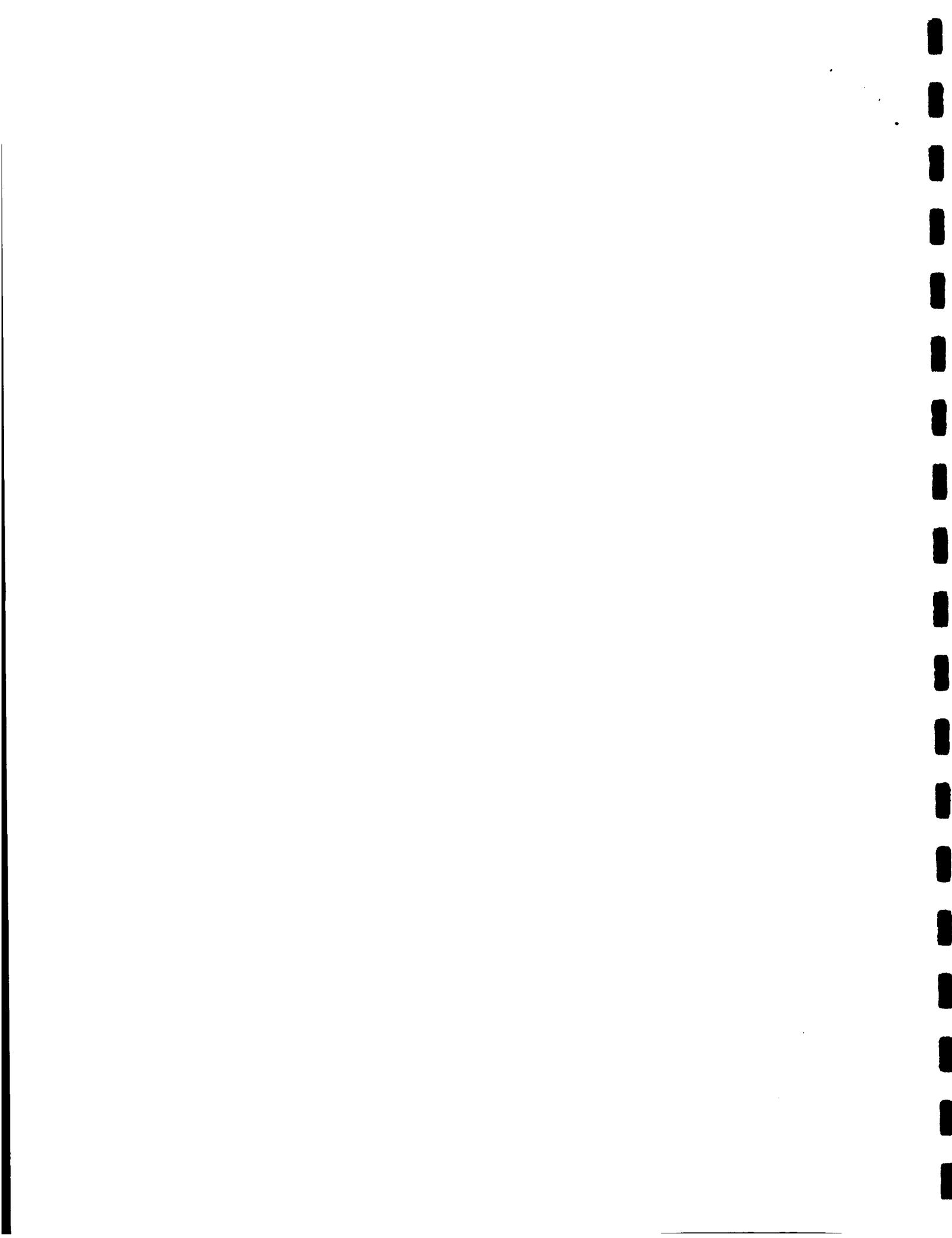


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SECTION

1.0 INTRODUCTION

This report is Volume 4 of the nine volumes of the final report on "Synthesis of Calculational Methods for the Design and Analysis of Radiation Shields for Nuclear Rocket Systems." Presented in this volume is a description of the neutron and gamma ray kernel attenuation program (KAP-V).

The KAP-V program is an integral part of both the "early" and "final" design radiation analysis methods provided for the Marshall Space Flight Center under this contract. A simplified schematic diagram of each method is shown in Figures 1 and 2. Both methods are fully described in Volume 1 of this report. The starting point for both methods is the POINT program (Volume 2) which prepares cross section and other basic data for use in the transport programs.

In the "early" design method (Figure 1), the TAPAT program system (Volume 3) computes one dimensional neutron and photon energy fluxes in the reactor geometry. From these fluxes, neutron and photon energy sources and distributions are obtained and are used as input to the KAP-V program. The KAP-V program (Volume 4) provides gamma ray and fast neutron radiation levels at locations external to the reactor. Radiation levels from the KAP-V program at a specific radial distance from the center of the reactor can then be employed in the TIC-TOC-TOE program (Volume 5) for calculating radiation quantities of interest in an on-axis liquid hydrogen propellant tank.

In the "final" design method (Figure 2), the ODD-K two dimensional transport program (Volume 6) provides neutron and photon energy fluxes throughout the reactor geometry. The NAGS data processing program (Volume 7) processes these fluxes and calculates neutron and photon radiation levels and neutron and photon energy sources within the reactor system. These sources can be employed in either the KAP-V program (Volume 4) or the FASTER Monte Carlo program (Volume 9) for obtaining radiation levels at locations external to the reactor system. In addition, the FASTER program can compute

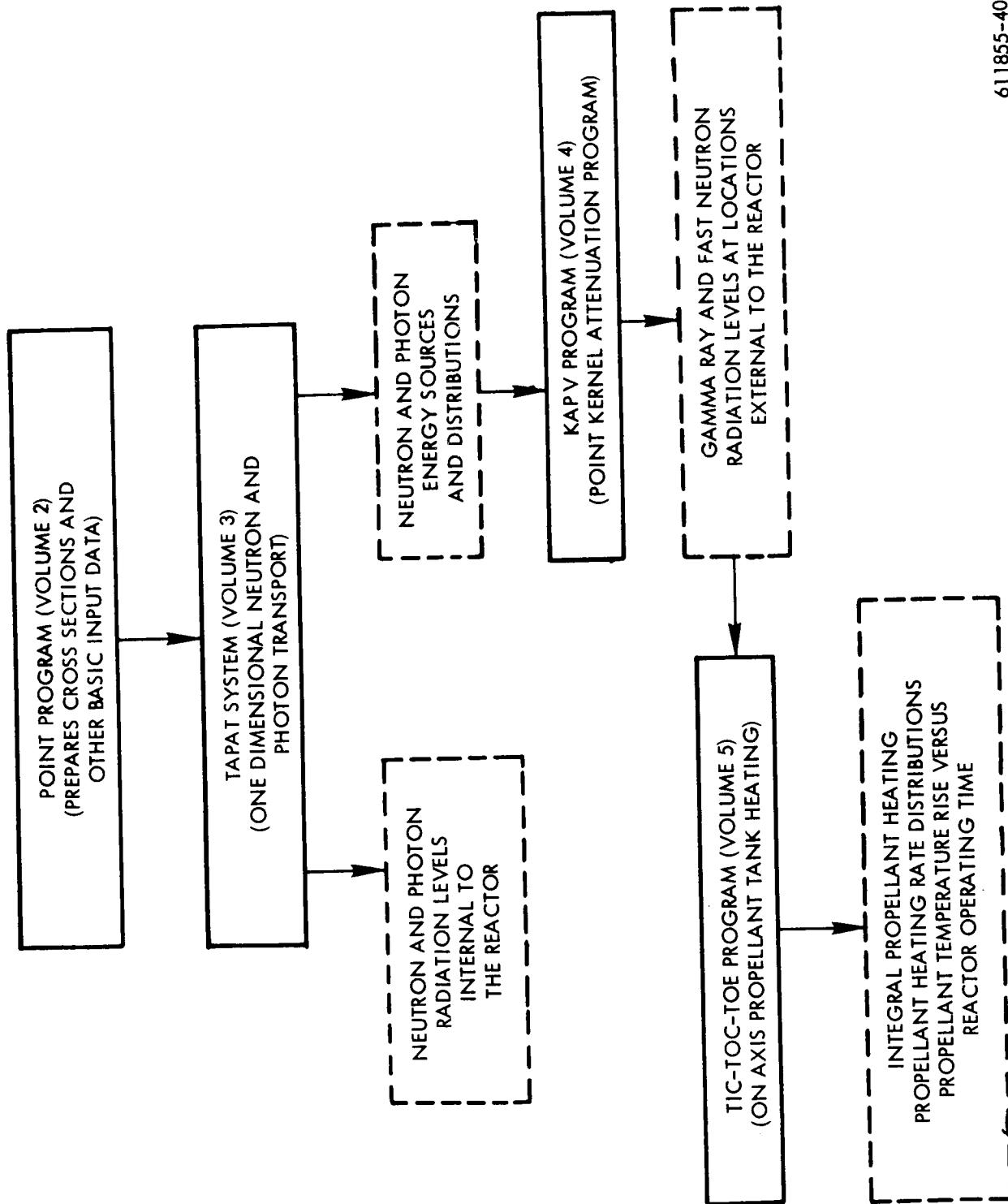


Figure 1. Schematic Diagram of the "Early" Design Method

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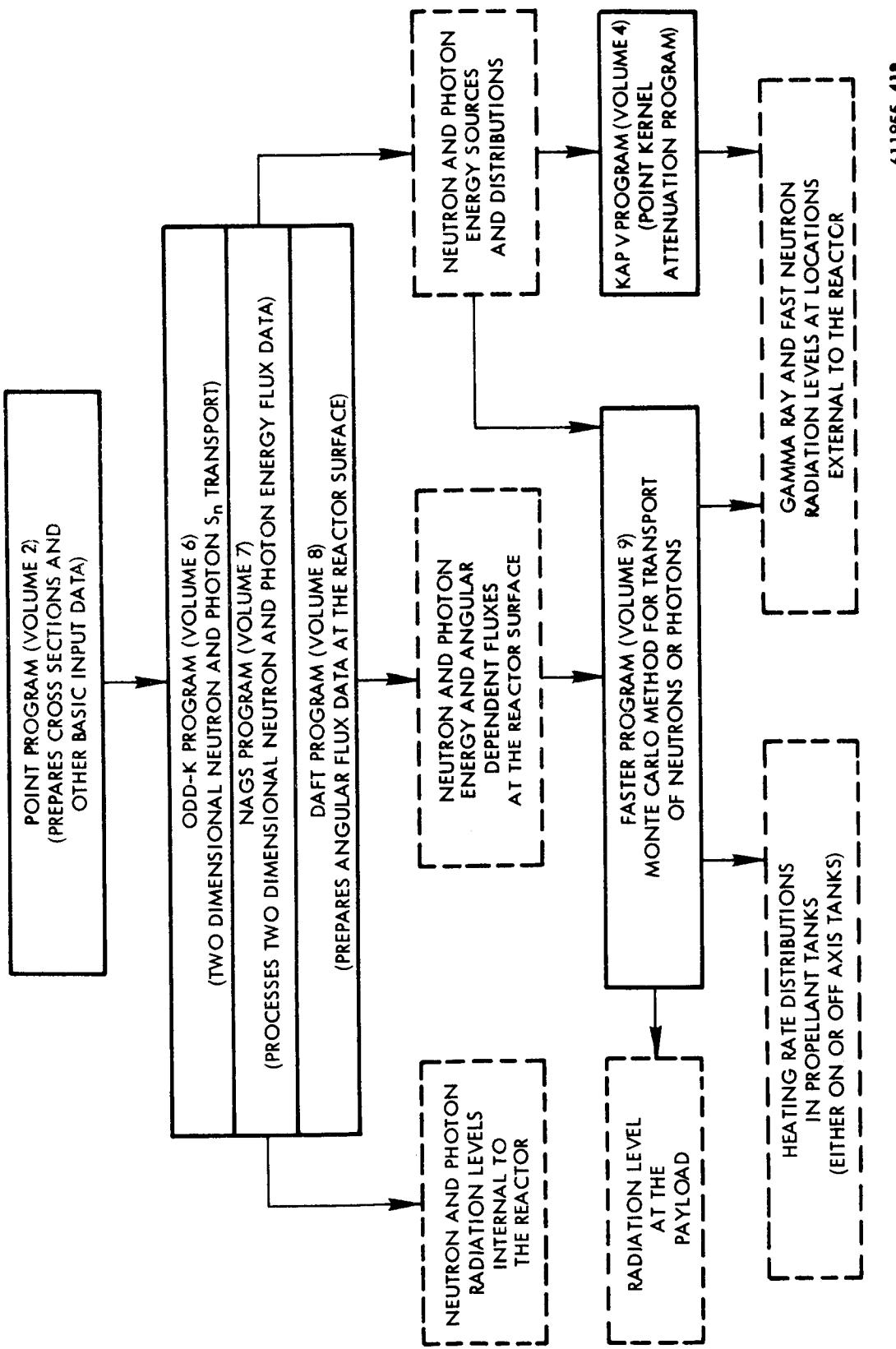


Figure 2. Schematic Diagram of the "Final" Design Method

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heating rate distributions in the liquid hydrogen propellant (in either an on- or an off-axis tank) and the radiation level at the payload. Alternately, the DAFT program (Volume 8) can prepare neutron and photon energy and angular dependent fluxes at the reactor surface from the ODD-K program for use in the FASTER Monte Carlo code.

Extensive radiation analysis at Westinghouse Astronuclear Laboratory indicated that a point kernel program was needed which included the desirable features of both the Los Alamos Scientific Laboratory QAD program series⁽¹⁾ and the General Electric Aircraft Nuclear Propulsion Department programs, 14-0 and 14-1⁽²⁾. This same conclusion was noted in a previous study⁽³⁾ performed for MSFC. The KAP-V program was formulated initially at WANL in FORTRAN II⁽⁴⁾, employing the QAD geometry routine as the only intact routine from the QAD IV program. Many features requested by KAP program users at WANL were incorporated in the program between 1963 and 1966, at which time work on the latest version, KAP-V, was initiated under the present contract. The entire programming effort on the KAP-V program was directed at producing a point kernel program which was a useful and efficient computer program in FORTRAN IV computer language. The present version of the KAP-V code is operational on the IBM 7094 Model II at WANL and MSFC, and on the CDC 6600 at the Westinghouse Telecomputer Center in Pittsburgh.

KAP-V is a point kernel program designed to calculate the radiation level at detector points located within or outside a complex radiation source geometry describable by a combination of quadratic surfaces. The program evaluates the material thicknesses intercepted along the line-of-sight from the source point to the detector point. These material thicknesses (or path lengths) then are employed in attenuation functions to calculate the flux, dose rate, or heating rate at the detector. The attenuation function for gamma rays employs exponential attenuation with a buildup factor. Three optional neutron attenuation functions are included: (1) a modified Albert-Welton function for calculating fast neutron dose rate using removal cross sections; (2) a bivariate polynomial expression for computing neutron spectra using infinite media moments data; and (3) a monovariate polynomial expression for computing neutron spectra using infinite media moments data.

The program also handles either cylindrical, spherical, disc, line, or point sources. Different source distributions may be employed for neutrons and gamma rays. A variety of options is available for describing the source distributions. The source distributions are assumed separable along the axis and radius of cylindrical-type source regions and independent of the azimuthal angular position for either spherical or cylindrical sources. An option is provided to describe azimuthal source density variation by specifying input data for discrete point sources.

Specific desirable features which have been incorporated in the KAP-V program are:

- (1) Input data preparation has been simplified to allow minimum input for running "stacked" cases.
- (2) The program uses the "point-in-region" concept to calculate the boundary surface-zone relationship ("ambiguity index") which is required as input in other point kernel codes.
- (3) A routine is included in the program to calculate gamma ray mass absorption coefficients for up to twenty elements as a function of input gamma ray energy which eliminates a great deal of the previously required input data.
- (4) A routine is included in the program to calculate the cubic polynomial coefficients for buildup factors as a function of input gamma ray energy from a library of bivariate polynomial data.
- (5) A routine is included which will interpolate a closely-spaced source distribution (obtained from a discrete ordinate transport or diffusion theory source calculation) to a source mesh description more amenable and economic to point kernel calculations.
- (6) A routine is included which calculates and normalizes point source strengths for a variety of source geometries and functional variations of source distributions.
- (7) Input data are checked for consistency to eliminate many erroneous calculations that can occur if input data for a problem is incomplete.
- (8) The program has the capability to calculate fluxes and/or other radiation responses such as heating rates at multiple detector points for each source region.

(9) The program has no set limit on the number of source regions which can be run in a single problem. This feature is handled as a set of stacked source region problems. The program computes the summation at each detector point of the neutron and/or photon radiation from each source region.

(10) The program allows the user to input separate source distributions for neutrons and gamma rays within the same source region.

(11) The program eliminates unnecessary response function computations by accumulating flux data as a function of detector point and group during the calculation for each source region. Calculations for up to ten response functions are performed only at the completion of each source region calculation and/or at the completion of source region problems.

(12) An option is included for calculating the flux at a detector located within a gamma ray source region. This option circumvents the numerical difficulties introduced by the "inverse square law," when a source point is too close to the detector.

The KAP-V program, with the above features, has proven to be an efficient tool for the analysis of the radiation environment produced by reactor sources. Comparisons, presented in Volume 1, between KAP calculations and NRX experimental radiation environment, and between Monte Carlo and transport calculations, give added confidence to point kernel analysis, provided that the code is judiciously employed.

The computer time required for running a "typical" problem is difficult to evaluate. The running time is dependent primarily upon the number of zones and associated boundaries along the path between each source and detector point. A complex problem run on the IBM 7094 computer required an average of 0.01 seconds per source point for each detector point. This complex geometry contained 44 zones and 32 boundaries.

Section 2 gives a more detailed description of the program, including the geometry, sources, and attenuation kernels. Section 3 briefly describes the logic of the KAP-V program. Input data instructions are presented in Section 4, and the output data is described in Section 5. A sample problem, along with an actual print-out of the results, is given in Section 6. The FORTRAN IV source program is listed in the Appendix.

SECTION

2.0 PROGRAM DESCRIPTION

2.1 GEOMETRY

The KAP-V program requires the following information in order to perform geometry and material calculations:

- 1) Zones (or regions) which are described by intersecting surfaces
- 2) Geometric surfaces which are described by various equations
- 3) Materials in the zones which are described by a material composition table
- 4) Nuclear properties of the materials.

Based on this input data, the KAP-V program calculates the "line-of-sight" distance (path length) through each material in each zone between each source point and the detector point.

Subsequent sections describe the techniques used in describing and solving geometry dependent quantities for a KAP-V problem.

2.1.1 Surfaces

The geometry of the problem can include the following types of equation surfaces:

- 1) Equations of a surface of revolution about any x, y, or z coordinate axis.
- 2) Equations of a plane normal to the x, y, or z axis of the reference system.
- 3) Equations of an elliptic cylinder about any z axis.
- 4) Equations of any quadrature surface by specifying appropriate equation coefficients.

To simplify the geometry input description, the program contains specific forms of the quadratic surface equations. Each of these equations are identified by an equation number. The equations available are as follows:

$$A(X^2) + B(Y^2) + C(Z^2) + X_0X + Y_0Y + Z_0Z - D = 0 \quad (\text{NEQBD} = 1)$$

$$A(X-X_0)^2 + B(Y-Y_0)^2 + C(Z-Z_0)^2 - D = 0 \quad (\text{NEQBD} = 2)$$

$$A(X-X_0)^2 + B(Y-Y_0)^2 - D = 0 \quad (\text{NEQBD} = 3)$$

$$X - D = 0 \quad (\text{NEQBD} = 4)$$

$$Y - D = 0 \quad (\text{NEQBD} = 5)$$
$$Z - D = 0 \quad (\text{NEQBD} = 6)$$

The quantities A, B, C, X_0 , Y_0 , Z_0 , and D are input parameters for the surfaces in a problem. The surface equation number defines the necessary parameters. The user specifies the equation number, NEQBD, and only those parameters necessary to solve the respective surface equation.

The surface defined by NEQBD = 4, 5, and 6 are planes normal to each of the coordinate axes. NEQBD = 3 is the equation for a cylindric surface with its axis parallel to the Z axis. NEQBD = 2 is an elliptic surface which, by specifying the A, B, and C coefficients properly, can describe elliptical cylindric surfaces with their axis parallel to each of the coordinate axes. NEQBD = 1 is a form of the general quadratic equation. By proper manipulation of the coefficient of a quadratic equation defining a surface, one can calculate the required coefficients A, B, C, D, X_0 , Y_0 , and Z_0 .

The equations shown above require that all parameters must be in units consistent with the nuclear properties of the zones.

The maximum number of surfaces that can be employed in a KAP-V problem is limited to 100.

2.1.2 Zones

A zone is defined as a region containing a homogeneous composition of materials and is bounded by a set of geometrical surfaces as defined by the surface equations. Geometrical surfaces described in a problem geometry are used to define the exterior boundaries of zones in a problem. Each zone is described as a volume bounded by as many as six intersecting surfaces. The boundary surfaces of a zone are designated by the geometric surface numbers.

KAP-V uses the "point-in-region" technique to assign the boundary surface-zone relationship values to each of the zone boundary numbers, since the relationship of the zone with respect to each of its boundary surfaces must be known for a KAP-V geometry calculation. This relationship is designated by the sign (plus or minus) of the zone boundary number and is called the "ambiguity index". The ambiguity index defines the position of a zone with respect to the zone

boundary surface as being an interior (+) or exterior (-) zone. In complex geometries, the assignment of ambiguity indices by the program user is difficult and time consuming. To circumvent this problem, the KAP-V program requires as input the Cartesian coordinates of a point (X_p , Y_p , Z_p) within each zone. Using these point coordinates, the designated surface numbers (LBD) for each boundary of a zone, and the equation number (NEQBD) of the surfaces, the calculation of the ambiguity index is straightforward. The surface equation and the coordinates (X_p , Y_p , Z_p) define the quantity, V, for each particular equation number (NEQBD = 1 through 6).

$$V = A(X_p - X_0)^2 + B(Y_p - Y_0)^2 + C(Z_p - Z_0)^2 + X_0 X_p + Y_0 Y_p + Z_0 Z_p - D \quad (\text{NEQBD} = 1)$$

$$V = A(X_p - X_0)^2 + B(Y_p - Y_0)^2 + C(Z_p - Z_0)^2 - D \quad (\text{NEQBD} = 2)$$

$$V = (X_p - X_0)^2 - (Y_p - Y_0)^2 - D \quad (\text{NEQBD} = 3)$$

$$V = X_p - D \quad (\text{NEQBD} = 4)$$

$$V = Y_p - D \quad (\text{NEQBD} = 5)$$

$$V = Z_p - D \quad (\text{NEQBD} = 6)$$

The sign (+) of the quantity V determines the ambiguity index of the boundary surface of the zone. This ambiguity index is assigned to the boundary number LBD. If V is negative, the zone is internal to the boundary surface and the boundary number LBD is given a positive sign. Similarly, if V is positive, the zone is external to the boundary surface and the boundary number LBD is given a negative sign. The ambiguity index calculation is performed at the beginning of each KAP-V source region calculation, and the computed signs are used for all geometry calculations for this source region.

External zones can be described by a single boundary surface. External boundary surfaces of external zones need not be defined. An external zone is recognized by the program if the sign of the boundary is a negative number. The sign of the boundary number of an external zone must be input by the program user.

2.1.3 Geometry Calculations

The geometry calculation begins with the computed Cartesian coordinates of a source

point (X_S , Y_S , Z_S) and a detector point (X_D , Y_D , Z_D). These coordinates are computed as follows:

$$X_S = \bar{R}_i \cos \bar{\theta}_{k,i}$$

$$Y_S = \bar{R}_i \sin \bar{\theta}_{k,i}$$

$$Z_S = \bar{z}_i$$

Cylindrical Source Point

or:

$$X_S = \bar{R}_i \cos \bar{\theta}_{k,i} \sin \bar{\phi}_i$$

$$Y_S = \bar{R}_i \sin \bar{\theta}_{k,i} \sin \bar{\phi}_i$$

$$Z_S = \bar{R}_i \cos \bar{\phi}_i$$

Spherical Source Point

$$X_D = R_D \cos \theta_D$$

$$Y_D = R_D \sin \theta_D$$

$$Z_D = Z_D$$

Detector Point

where:

\bar{R}_i = the geometrical mean of the source interval bounded by the radii R_i and R_{i+1} , i.e.,

$$\bar{R}_i = \sqrt{\frac{R_{i+1}^2 + R_i^2}{2}}$$

$\bar{\theta}_{k,i}$ = the arithmetic mean of the azimuthal source interval for each radial interval, i , bounded by $\theta_{k,i}$ and $\theta_{k+1,i}$, i.e.,

$$\bar{\theta}_{k,i} = \frac{\theta_{k+1,i} + \theta_{k,i}}{2}$$

\bar{Z}_i = the arithmetic mean of the axial source interval bounded by Z_i and Z_{i+1} , i.e.,

$$\bar{Z}_i = \frac{Z_{i+1} + Z_i}{2}$$

$\bar{\phi}_i$ = the arithmetic mean of the polar source interval bounded by ϕ_i and ϕ_{i+1} , i.e.,

$$\bar{\phi}_i = \frac{\phi_{i+1} + \phi_i}{2}$$

R_D = the radial coordinate of the detector point (input to the problem)

θ_D = the azimuthal coordinate of the detector point (input to the problem)

Z_D = the axial coordinate of the detector point (input to the problem)

The total "line-of-sight" distance, ρ , between a source point and a detector point, and the direction cosines (α, β, γ) are then computed as follows:

$$\rho = \sqrt{(X_D - X_S)^2 + (Y_D - Y_S)^2 + (Z_D - Z_S)^2}$$

$$\alpha = \frac{X_D - X_S}{\rho}$$

$$\beta = \frac{Y_D - Y_S}{\rho}$$

$$\gamma = \frac{Z_D - Z_S}{\rho}$$

The next step in the geometry calculation is to obtain the path length, ρ_Z , traversed in each region along the "line-of-sight". This calculation begins with the coordinates of a "pseudo-point" (X', Y', Z'), along the "line-of-sight" which is related to the original source point by the input value, FUDGE, designated by Δ . This calculation is performed as:

$$X' = X_S + \alpha \Delta$$

$$Y' = Y_S + \beta \Delta$$

$$Z' = Z_S + \gamma \Delta$$

This pseudo-point, (X', Y', Z') , is used in conjunction with input zone boundaries, surface numbers, surface equations, input surface parameters, and the source zone number to calculate the correct zone in which X' , Y' , and Z' lies. The actual operation performed is a cyclic calculation of the quantities, $V_{bZ'}$, for each boundary, b , of the source zone, Z . The cyclic calculation begins in the zone specified by the input value, IZSO. The values of $V_{bZ'}$ depend on the equation number NEQBD _{b} of boundary b , and follow as:

$$V_{bZ'} = A(X_p - X_0)^2 + B(Y_p - Y_0)^2 + C(Z_p - Z_0)^2 + X_0 X_p + Y_0 Y_p + Z_0 Z_p - D,$$

$$V_{bZ'} = A(X_p - X_0)^2 + B(Y_p - Y_0)^2 + C(Z_p - Z_0)^2 - D,$$

$$V_{bZ'} = (X_p - X_0)^2 - (Y_p - Y_0)^2 - D,$$

$$V_{bZ'} = X_p - D,$$

$$V_{bZ'} = Y_p - D,$$

$$V_{bZ'} = Z_p - D.$$

If the sign of the quantity, $V_{bZ'}$, and the sign (ambiguity index) of the boundary surface number LBD _{bZ'} are of opposite sign for all boundary surfaces, the point (X', Y', Z') lies within the region or zone, Z . If the point does not lie in the IZSO zone, the program searches the zones in a specific order as follows: IZSO + 1, IZSO + 2,up to the number of zones, NREG; then it begins with Zone 1, 2, etc. up to IZSO-1. If a zone is found which contains the point (X', Y', Z') , the calculation proceeds to the next geometry calculation step. If no zone can be found which contains the point, the region calculation is terminated by printing an error statement along with the results for source regions preceding that one in which the error occurred.

The next step in calculating the path length in each region involves the analytic solution of distances from the point (X', Y', Z') to each boundary surface of the zone. The solution is obtained by solving the boundary equations for the point of intersection of the "line-of-sight" and the surface in question. These distances to each boundary are sequentially tested, and the minimum distance in the correct direction is selected as the distance from the "pseudo-point", (X', Y', Z') to the correct boundary. This quantity is defined as ρ_Z' .

At this point in the calculation, the correct path length in the zone is calculated as:

$$\rho_Z' = \rho_Z + \Delta$$

The material path lengths, ρ_m , for each material, m , are immediately calculated as cumulative sums from ρ_Z' , and composition material matrix values, θ_{mc} , as follows:

$$\rho_m = \rho_m + \rho_Z' \cdot \theta_{m,c}$$

In the above equation, c is the specific composition of the zone, Z , (quantities, $\theta_{m,c}$, are discussed in the next section), and ρ_m is set to zero at the beginning of each source-to-detector calculation. The final operation in the source zone path length calculation is the starting point for obtaining the next zone (along the line-of-sight) path length. The input values, NTRYZN_{bZ'}, determine the "most probable" zone entered upon crossing boundary, b , of the zone, Z . With the last calculated value of ρ_Z' , a new "pseudo-point" along the line-of-sight is calculated as:

$$X' = X' + \alpha \rho_Z'$$

$$Y' = Y' + \beta \rho_Z'$$

$$Z' = Z' + \gamma \rho_Z'$$

These current "pseudo-point" coordinates and the zone number, NTRYZN_{bZ'}, are used in the operations described above in calculating data for the next zone traversed in the source-detector "line-of-sight". The data of:

- 1) the correct zone,
- 2) the zone path lengths,
- 3) the new material path lengths, and
- 4) the zone entered upon crossing the correct boundary

are obtained for each zone along the line-of-sight. This cyclic procedure (calculation of zone path length) continues until an "outside zone" is reached or until the detector point, (X_D' , Y_D' , Z_D'), is reached. At this point in the program, the total material path lengths,

ρ_m , on this particular source-to-detector "line-of-sight" are known, and hence the material attenuation functions and the source point detector flux can be evaluated. This source point-to-detector point calculation is repeated for each source point in a source region until the entire source region is accounted for.

2.2 MATERIAL AND NUCLEAR PROPERTIES

The contents of the KAP-V regions are described as homogeneous mixtures of constituents (elements or materials). These elements or materials, which may be used in as many regions as desired, are defined by a matrix of numbers, $\theta_{m,c}$. The quantities, $\theta_{m,c}$, describe either the density or the volume fraction of each constituent, m , in each composition, c . The density (weight of material/region volume) is used, for example, if the mass gamma ray absorption coefficients in units of cm^2/gm are employed. Volume fractions, i.e., the fraction of the region volume occupied by an element or material in a region composition is used, for example, if linear gamma ray absorption coefficients in units of cm^{-1} are input to the program. The maximum number of compositions is limited to fifty, each consisting of a maximum of 20 constituents (elements or materials). An example of this matrix is shown in Table 1.

The data in the matrix, $\theta_{m,c}$, are used in the geometry calculation to obtain the total thickness density, gm/cm^2 , if $\theta_{m,c}$ is in units of gm/cm^3 , or total thickness, cm , if $\theta_{m,c}$ is input as volume fractions. This operation is as follows:

$$\rho_m = \sum_Z \theta_{m,c} \cdot \rho_Z$$

where the composition number, c , is specified as being in zone, Z .

Values of ρ_m are used in calculating the neutron and/or groupwise gamma ray penetration depth for each source to detector path length through each element or material, m .

A "void" is defined as a composition with zero density or volume fraction for each constituent in the matrix.

Nuclear data, such as gamma ray attenuation coefficients in a KAP-V problem are required for each constituent (element or material) in the material matrix. Specifically, required nuclear properties are the groupwise gamma ray mass or linear absorption coefficients,

TABLE 1
ELEMENT OR MATERIAL COMPOSITION MATRIX ($\Theta_{M,C}$)

Composition Number, C	Element or Material Number, M →										$\Theta_{MAT, NCOMP}$
	1	2	3	4	MAT	
1	$\Theta_{1,1}$	$\Theta_{2,1}$	$\Theta_{3,1}$	$\Theta_{4,1}$	$\Theta_{MAT, 1}$
2	$\Theta_{1,2}$	$\Theta_{2,2}$	$\Theta_{3,2}$	$\Theta_{4,2}$	$\Theta_{MAT, 2}$
3	$\Theta_{1,3}$	$\Theta_{2,3}$	$\Theta_{3,3}$	$\Theta_{4,3}$	$\Theta_{MAT, 3}$
.
.
.
.
.
NCOMP	$\Theta_{1, NCOMP}$		$\Theta_{3, NCOMP}$		
		$\Theta_{2, NCOMP}$		$\Theta_{4, NCOMP}$							

$(\mu/\rho)_m$, or μ_m , and the mass or linear neutron removal cross sections, $(\Sigma/\rho)_m$ or Σ_m . The user must input (or compile from the library as discussed in Section 2.6) these quantities in dimensions consistent with the material matrix quantities, $\theta_{m,c}$. For example, if $(\mu/\rho)_m$ and $(\Sigma/\rho)_m$ are input in units of cm^2/gram , then $\theta_{m,c}$'s must be in units of grams/cm^3 .

2.3 SOURCE

The KAP-V program can employ either a cylindrical or a spherical source region, as well as the basic source geometries of a point, line, or disk. The source distribution in cylindrical or spherical geometry is assumed separable in the spatial (radial, axial, or polar, and azimuthal) coordinates. The source energy distribution is input as a separate quantity.

The source distribution can be input as unnormalized radial and/or axial source data. The program integrates and normalizes the input distribution data to obtain the source strength of the source point representing each finite source volume in the source region.

The program assumes that the azimuthal distribution of the source density is uniform. The uniform azimuthal distribution data is used within the program to properly normalize the source.

The same distribution may be assumed for the gamma ray and neutron source, or, the user may specify a different source distribution and normalization constant for gamma rays and neutrons.

2.3.1 Energy Distribution

The quantity, $\Gamma(E_n \text{ or } E_G)$, defines the source strength in each energy group. For the gamma ray source strength, the quantity $\Gamma(E_G)$, may represent the number of particles (or photons) of energy, E_G , or energy release (Mev) at energy E_G . The user must provide dimensionally consistent data for the total power (if source strengths are input on a per watt basis or the gamma ray source input), $A_{T,I'}$ in the source region, along with the gamma ray group source strengths, $\Gamma(E_G)$.

The neutron energy distribution parameters, $\Gamma(E_n)$, for the neutron differential energy spectrum function can be input as integration factors, ΔE_n , or as parameters to convert the neutron spectra data from units of one fission source neutron to units of neutrons per fission or

per watt. Also the quantity, $\Gamma(E_n)$, allows the user to input the total power or total fission rate in order to make the spectral data dimensionally consistent. The units of $\Gamma(E_n)$ must be dimensionally consistent with the total power (or neutron source strength) input as $A_{T'2}$.

Although the Albert-Welton function computes an energy independent dose rate based on a fission spectrum source, a separate quantity, Γ_{AW} , may be input to the code to provide a "source strength" for use with this function.

2.3.2 Spatial Distribution

The spatial source in a KAP-V source region is represented as a finite number of volume elements, each of which are represented as a source point. The source density at each source point, and the location of each source point, are determined by the program from input source parameters. The program includes techniques to calculate source point densities from: (1) analytical functions (uniform or flat, cosine, exponential), or (2) pointwise source values. The unnormalized source densities for gamma rays or neutrons are defined as the separable quantities: $f(\bar{R}_i)$, $f(\bar{Z}_i)$, $f(\bar{\phi}_i)$, and $f(\bar{\theta}_{k,i})$, for the space coordinates of R (radius for cylindrical or spherical sources), Z (axial dimension for cylindrical sources), ϕ (polar angle for spherical sources), and θ (azimuthal angle for cylindrical and spherical sources), respectively.

By means of the input quantities, (ISRC, ISZC, and ISTC), the program user selects a technique for calculating the source density variation of interest.

Analytical Functions

The KAP-V cylindrical and spherical source density functions for the variable, R (radius), are described in equations 2.4 through 2.7.

The function, $f(\bar{R}_i)$, in equations 2.4 through 2.7 defines the unnormalized source density for each radial interval. The source point is placed at \bar{R}_i of each annular source interval bounded by R_i and R_{i+1} . The quantity, \bar{R}_i , is defined by equation 2.1.

$$\bar{R}_i = \sqrt{\frac{R_{i+1}^2 + R_i^2}{2}} \quad (2.1)$$

The function $f(\bar{Z}_i \text{ or } \bar{\theta}_i)$ in equation 2.8 through 2.11 defines the unnormalized source for each axial or polar interval. The source point is placed at the arithmetic mean, \bar{Z}_i , or $\bar{\theta}_i$, of the axial or polar source interval bounded by Z_i and Z_{i+1} or θ_i and θ_{i+1} as defined by equations 2.2 and 2.3:

$$\bar{Z}_i = \frac{Z_{i+1} + Z_i}{2} \quad (2.2)$$

$$\bar{\theta}_i = \frac{\theta_{i+1} + \theta_i}{2} \quad (2.3)$$

The analytical functions for the cylinder and sphere, and the only required input parameters, follow as:

Cylinder - Uniform Source Density (ISRC = 1)

$$f(\bar{R}_i) = \int_{R_i}^{R_{i+1}} R dR \quad (2.4)$$

where values of R_i are required input.

Cylinder - Cosine Variation of Source Density (ISRC = 2)

$$f(\bar{R}_i) = \int_{R_i}^{R_{i+1}} \cos x_1 [R - x_2] R dR \quad (2.5)$$

where values of x_1 , x_2 and R_i are required input.

Cylinder - Exponential Variation of Source Density (ISRC = 4)

$$f(\bar{R}_i) = \int_{R_i}^{R_{i+1}} x_2 \cdot \exp [x_1 R] R dR \quad (2.6)$$

where values of x_1 , x_2 and R_i are required input.

Sphere - Uniform Source Density (ISRC = 6)

$$f(\bar{R}_i) = \int_{R_i}^{R_{i+1}} R^2 dR \quad (2.7)$$

where values of R_i are required input.

Cylinder - Uniform Source Density (ISZC = 1)

$$f(\bar{Z}_i) = \int_{Z_i}^{Z_{i+1}} dz \quad (2.8)$$

where values of Z_i are required input.

Cylinder - Cosine Variation of Source Density (ISZC = 2)

$$f(\bar{Z}_i) = \int_{Z_i}^{Z_{i+1}} \cos \xi_1 [Z - \xi_2] dz \quad (2.9)$$

where values of ξ_1 , ξ_2 and Z_i are required input.

Cylinder - Exponential Variation of Source Density (ISZC = 4)

$$f(\bar{Z}_i) = \int_{Z_i}^{Z_{i+1}} \xi_2 \cdot \exp [\xi_1 Z] dz \quad (2.10)$$

where values of ξ_1 , ξ_2 and Z_i are required input.

Spherical - Uniform Variation of Source Density (ISZC = 6)

$$f(\bar{\phi}_i) = \int_{\phi_i}^{\phi_{i+1}} \cos \phi d\phi \quad (2.11)$$

where values of ϕ_i are required input.

Pointwise Source Density

Pointwise source density can be specified as input point values at the spatial limits (R_i , R_{i+1} or Z_i , Z_{i+1}) of each interval in the source region, or as point values calculated from input point values at spatial points other than the desired space points.

Input point values defined as $g(R_i)$ at R_i and $g(Z_i)$ at Z_i are used directly in the program to obtain the source density parameters of $f(\bar{R}_i)$ and $f(\bar{Z}_i)$. The use of these point values are specified by the input values ISRC and ISZC. The techniques used for direct input value calculations are based on a source variation between two adjacent spatial points, i.e., R_i , R_{i+1} or Z_i , Z_{i+1} . The point value calculation for cylindrical and spherical source regions follow as:

Cylinder - Linear Variation of Source Density (ISRC = 3)

$$f(\bar{R}_i) = \int_{R_i}^{R_{i+1}} [aR + b] R dR \quad (2.12)$$

where a and b are computed internally by the program from the adjacent values $g(R_i)$ at R_i and $g(R_{i+1})$ at R_{i+1} .

Cylinder - Exponential Variation of Source Density (ISRC = 5)

$$f(\bar{R}_i) = \int_{R_i}^{R_{i+1}} x_2 \cdot \exp[x_1 R] R dR \quad (2.13)$$

where x_1 and x_2 are computed internally by the program from the adjacent values $g(R_i)$ at R_i and $g(R_{i+1})$ at R_{i+1} .

Sphere - Linear Variation of Source Density (ISRC = 7)

$$f(\bar{R}_i) = \int_{R_i}^{R_{i+1}} [aR + b] R^2 dR \quad (2.14)$$

where a and b are computed internally by the program from the adjacent values $g(R_i)$ at R_i and $g(R_{i+1})$ at R_{i+1} .

Cylinder - Linear Variation of Source Density (ISZC = 3)

$$f(\bar{Z}_i) = \int_{Z_i}^{Z_{i+1}} [aZ + b] dZ \quad (2.15)$$

where a and b are computed internally by the program from the adjacent values $g(Z_i)$ at Z_i and $g(Z_{i+1})$ at Z_{i+1} .

Cylinder - Exponential Variation of Source Density (ISZC = 5)

$$f(\bar{Z}_i) = \int_{Z_i}^{Z_{i+1}} \xi_2 \cdot \exp [\xi_1 Z] dZ \quad (2.16)$$

where ξ_1 and ξ_2 are computed internally by the program from the adjacent values $g(Z_i)$ at Z_i and $g(Z_{i+1})$ at Z_{i+1} .

The technique of calculating point values from input point values at spatial points other than the desired space points is a very useful facet of the KAP V program. The input point values may be representative of a fine radial mesh output from a transport code problem. The fine mesh may, however, provide too many source points for economical use in a point kernel calculation. Therefore, the user of the KAP-V code can input the exact transport code output data, of $g'(R')$ versus R' , and the code will interpolate new point values $g(R)$ at R , where the values of the new radial mesh, R , are selected to better represent a point kernel source point description. The point values $g(R)$ at R are used in the equations 2.12 through 2.16 described above to calculate the pointwise source density. This interpolation technique is controlled by the input quantity ISIT, and the spatial distributions of the source are input as the point values, FSIT, at RSIT and ZSIT, as described in the section on input data, Section 4.0.

Azimuthal Variation

The azimuthal distribution is assumed to be uniform in the azimuthal space variable, θ , for cylindrical and spherical source regions. The user of the KAP-V program has the option of specifying the mode in which the routine will subdivide the θ variable into intervals as shown in equations 2.17 through 2.19.

Same iNumber of Azimuthal Intervals in All Radial Intervals (ISTC = 1)

$$f(\bar{\theta}_{k,i}) = \int_{\theta_{k,i}}^{\theta_{k+1,i}} d\theta \quad \text{Required input, } \theta_{k,1} \quad (2.17)$$

Variable Number of Azimuthal Intervals in Each Radial Interval (ISTC = 2)

$$f(\bar{\theta}_{k,i}) = \int_{\theta_{k,i}}^{\theta_{k+1,i}} d\theta \quad \theta_{k,i} \text{ are required input for each radial interval} \quad (2.18)$$

Variable Number of Azimuthal Intervals in Each Radial Interval (ISTC = 3)

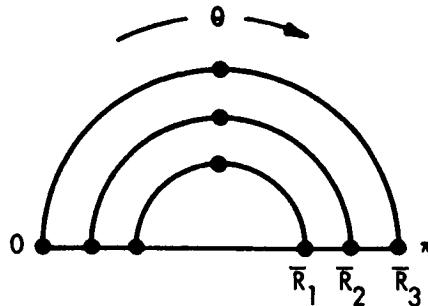
$$f(\bar{\theta}_{k,i}) = \frac{\int_{\theta_{k,1}}^{\theta_{k+1,i}} d\theta}{k_i} \quad k_i \text{ is required input for radial intervals} \quad (2.19)$$

These three options allow description of the three possible variations shown in Figure 3. One additional option is available to the program user: if discrete point sources are of interest, then the quantity ISRC is input as a zero, and the source density is input as $f(\bar{R}, \bar{Z}, \bar{\theta})$. This option allows the user to describe a variable source density in the azimuthal variable, θ .

2.3.3 Normalization of Source Parameters

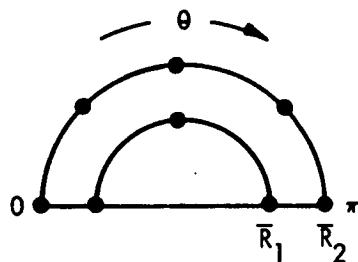
The source normalization routine in the KAP-V program gives the user the versatility to input the source distribution data in unnormalized form. The program normalizes the source parameters as shown in the equations which follow.

SOURCE GEOMETRY

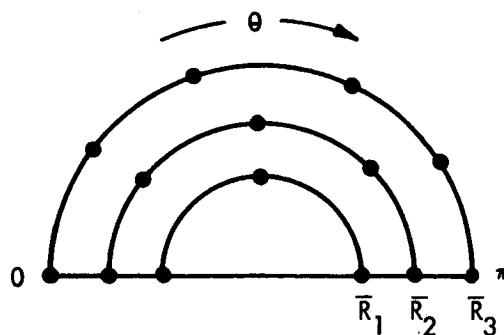


INPUT DATA

IF ISTC = 1, $\theta_{K,1}$ IS INPUT AS 0.0,
1.5708, 3.1416



IF ISTC = 2, $\theta_{K,1}$ IS INPUT AS 0,
1.5708, 3.1416, $\theta_{K,2}$ IS INPUT AS 0,
.7854, 1.5708, 2.3562, 3.1416



IF ISTC = 3,
 K_1 IS INPUT AS 2,
 K_2 IS INPUT AS 4,
 K_3 IS INPUT AS 5

611855-44B

Figure 3. Examples of Azimuthal Source Description

Cylindrical Source Region
Gamma Ray Source

$$P'_1 = \frac{A_{T,1}}{\left[\theta_{1,k+1} - \theta_{1,1} \right] \cdot \iint_R Z f_1(\bar{R}_i) \cdot f_1(\bar{Z}_i) \cdot R dR dZ} \quad (2.20)$$

$$f'_1(\bar{\theta}_{k,i}) = P'_1 \cdot f_1(\bar{\theta}_{k,i}) \quad (2.21)$$

where:

$A_{T,1}$ = the gamma ray source normalization constant which is input as ASOI(1).

$\theta_{1,k+1}$ = the upper limit of the θ variable for the gamma ray problem

$\theta_{1,1}$ = the lower limit of the θ variable for the gamma ray problem

$f_1(\bar{R}_i)$ = the unnormalized gamma ray source density at each radius, R_i

$f_1(\bar{Z}_i)$ = the unnormalized gamma ray source density at each value of Z_i

$f_1(\bar{\theta}_{k,i})$ = the unnormalized gamma ray source density at each azimuthal value θ_k for each radial position, i (i.e., $\Delta\theta_{k,i}$)

Neutron Source

If ASOI(2) is input as zero, then the program uses the gamma ray source, $A_{T,1}$, and distributions for neutrons and solves the same equation described for gamma rays. If ASOI(2) is not input as zero, then the program solves the following equations:

$$P'_2 = \frac{A_{T,2}}{\left[\theta_{1,k+1} - \theta_{1,1} \right] \cdot \iint_{R Z} f_2(\bar{R}_i) \cdot f_2(\bar{Z}_i) R dR dZ} \quad (2.22)$$

$$f'_2(\bar{\theta}_{k,i}) = P'_2 \cdot f_2(\bar{\theta}_{k,i}) \quad (2.23)$$

where all terms were previously defined and the subscript 2 refers to neutron data.

The normalized quantity, $f'(\bar{\theta}_{k,i})$, and the unnormalized quantities, $f(\bar{R}_i)$ and $f(\bar{Z}_i)$, provide the source magnitude of the source point defined at the coordinates \bar{R}_i , \bar{Z}_i , and $\bar{\theta}_{k,i}$ as $S(\bar{R}_i, \bar{Z}_i, \bar{\theta}_{k,i})$ in units of watts or particles per second.

This source parameter data is stored internally to the program and is used in the definition of the source magnitude for each source-to-detector calculation.

The user has the option of specifying the source region azimuthal parameters, $\theta_{1,k+1}$, $\theta_{1,1}$, or $\theta_{2,k+1}$, $\theta_{2,1}$ such that the source region symmetry is accounted for in the normalization. If $A_{T,1}$ or $A_{T,2}$ is total power (or source strength), and if $\theta_{1,k+1}$ and $\theta_{1,1}$ are π and 0.0, the power or source density is effectively twice that in the source region. Hence each source point source at $\theta_{1,k}$ includes its mirror image at $\theta_{1,k+\pi}$.

Spherical Source Region

Gamma Ray Source

$$P'_1 = \frac{A_{T,1}}{\left[\theta_{1,k+1} - \theta_{1,1} \right] \cdot \iint_{\phi R} \sin(\phi) \cdot f_1(\bar{\phi}_i) \cdot f_1(\bar{R}_i) R^2 dR d\phi} \quad (2.24)$$

$$f'_1(\bar{\theta}_{k,i}) = P'_1 \cdot f_1(\bar{\theta}_{k,i}) \quad (2.25)$$

where:

$f_1(\bar{\phi}_i)$ = the unnormalized gamma ray source density at each polar coordinate, $\bar{\phi}_i$ and all other terms are as previously defined.

Neutron Source

$$P_2' = \frac{A_{T,2}}{\left[\theta_{1,k+1} - \theta_{1,1} \right] \cdot \int \int_{\phi, R} \sin(\phi) \cdot f_2(\bar{\theta}_i) \cdot f_2(\bar{R}_i) R^2 dR d\phi} \quad (2.26)$$

$$f_2(\bar{\theta}_{k,i}) = P_2' \cdot f_2(\bar{\theta}_{k,i}) \quad (2.27)$$

where all terms are as previously defined, and the subscript 2 refers to neutron data.

2.4 MATERIAL ATTENUATION

2.4.1 Fast Neutron

The KAP V program contains three material attenuation options for obtaining the neutron response at a detector point. The user can specify the use of the modified Albert-Welton material attenuation function⁽²⁾, and either a bivariant or monovariant polynomial material attenuation function for calculating differential neutron spectra employing infinite media moments data. Various conversion factors can be applied to any of these three functions, as described in Section 2.5 for conversion to different units.

Fast Neutron Dose Rate

A modification of the Albert-Welton function is used to calculate the fast neutron dose rate from fission sources in mixtures of hydrogenous and heavy materials. This function combines a theoretical hydrogen cross section with integration over the fission neutron spectrum to obtain the uncollided flux or dose. Attenuation effects of non-hydrogenous materials are included by employing exponential attenuation with effective removal cross sections.

The basic assumption in the Albert-Welton function is that all heavy materials are followed by sufficient hydrogenous materials to validate the use of neutron removal cross sections for the heavy materials. In addition, the Albert-Welton function is an integral quantity calculated from theoretical consideration of the energy dependence of neutron cross sections and the variation of neutron spectrum with penetration in hydrogen. Hence, the Albert-Welton function dose rate does not include buildup of neutrons at lower energies.

The equation coded in the program for computing the energy independent fast neutron response at the detector from each source point is:

$$D_n = \frac{r_{AW} \cdot S(\bar{R}_i, \bar{Z}_i, \bar{\theta}_k) \cdot \psi(W'_R, X_R)}{4\pi \rho^2} \quad (2.28)$$

where:

r_{AW} = the "source strength" for use in the Albert-Welton function.

$S(\bar{R}_i, \bar{Z}_i, \bar{\theta}_k)$ = the volume weighted source strength for the source point, located at $\bar{R}_i, \bar{Z}_i, \bar{\theta}_k, i$

ρ = the distance (cm) from the source point to the detector point.

$$\psi(W'_R, X_R) = \alpha_1 \cdot [X_R]^{\alpha_2} \cdot \exp[-\alpha_3 X_R]^{\alpha_4} \cdot \exp[-W'_R] \quad (2.29)$$

$$W'_R = \sum_{m=1}^M \Sigma_m' \rho_m \quad (2.30)$$

$$X_R = \sum_{m=1}^M \eta_m \rho_m \quad (2.31)$$

Σ_m' = non-hydrogenous removal cross section for all materials, m

ρ_m = the path length for each material, m

η_m = the ratio of the hydrogen density in material m to the hydrogen density in water

$\alpha_1, \alpha_2, \alpha_3$, and α_4 are constants.

The user will note that in equation 2.31 the units of X_R may be either cm or gm/cm² depending on the units of the path length (ρ_m). Hence, care must be exercised in applying the input quantity, η_m , to assure proper units.

Since the Albert-Welton function (equation 2.29) cannot be used to calculate the neutron dose rate for small values of X_R , the following equation (originally coded in Program 14-0) is employed in the KAP V code:

$$\Psi(W'_R, X_R) = \alpha_5 \cdot \exp[-\alpha_7 X_R] \cdot \exp[-W'_R], \text{ if } X_R < \alpha_6 \quad (2.32)$$

Values of α_5 , α_6 , and α_7 , are input to the code. The units of α_1 , (eq. 2.29) and α_5 (eq. 2.32) must be compatible to provide proper units for dose rate calculations.

Neutron Spectra

Either a monovariant or a bivariant polynomial expression can be employed to calculate the differential neutron spectrum. The monovariant polynomial data are available as sets of data representing specific differential spectrum energy points and are solved only in the variable of material depth penetration, W_R . In contrast, the bivariant polynomial data are available as sets of data which are solved as a function of initial neutron energy, E_n , as specified by the user, and material depth penetration, W_R .

The monovariant and bivariant polynomial representation of the moments method data is derived from the infinite medium moments method data such as that generated by the Nuclear Development Corporation (7). The polynomial coefficients are applicable over specific depth penetration or energy. The user of the KAP program specifies the applicable ranges of polynomial data. Both polynomials are based on the infinite medium of the particular material (hereafter called the reference material) used in the moments method calculation. The inclusion of other materials is based on their equivalent neutron removal in comparison to the reference material; hence, extreme care must be used in selecting the removal cross section for material in non-hydrogenous media (e.g., carbon and beryllium media). The equivalent depth penetration in the reference material is calculated in the program as:

$$W_R = \sum_{m=1}^M \Sigma_m \rho / \Sigma_R \quad (2.33)$$

where, Σ_R is the neutron removal cross section for this reference material.

The user specifies a separate set of neutron removal cross sections, Σ_m , for the neutron spectra option which is not the same set that was used in the Albert-Welton function. The program then uses these cross section data in the evaluation of W_R . A restriction of the moments data evaluation in the KAP program arises when the depth penetration exceeds the range of applicability of the polynomial data. The program automatically truncates the polynomial evaluation at $W_R = 120 \text{ gm/cm}^2$. However, the user is provided with the capability to input energy dependent extrapolation parameters, $\lambda(E_n)$, for extending the range of the moments data functions.

The equation coded in the program for computing the differential neutron energy response at the detector from each source point for each scattered neutron energy, E_n is:

$$D_n(E_n) = \frac{\Gamma(E_n) \cdot S(\bar{R}_i, \bar{Z}_i, \bar{G}_k) \cdot \Psi(W_R, E_n)}{4\pi\rho^2} \quad (2.34)$$

where:

$\Gamma(E_n)$ = the source strength for each group, E_n , and all other parameters were previously defined except $\Psi(W_R, E_n)$ which is described below.

Neutron Spectra Monovariant Polynomial

$$\Psi(W_R, E_n) = \exp \left[f(W_R, E_n) \right] \quad (2.35)$$

where:

$$f(W_R, E_n) = \sum_{i=0}^4 \gamma_i(E_n) \cdot W_R^i \quad (2.36)$$

γ_i = monovariant polynomial coefficients fit to the infinite media neutron spectra data

If, $W_R > 120.0 \text{ gm/cm}^2$,

$$f(W_R, E_n) = \left[\sum_{i=0}^4 \gamma_i(E_n) \cdot 120.0^i \right] \cdot \exp \left[-\lambda(E_n) \cdot (W_R - 120.0) \right] \quad (2.37)$$

$\lambda(E_n)$ = parameter for extrapolation of the neutron spectra data for each neutron energy, E_n

Neutron Spectra Bivariant Polynomial

$$\Psi(W_R, E_n) = \exp \left[f(W_R, E_n) \right] \quad (2.38)$$

where:

$$f(W_R, E_n) = \sum_{i=0}^4 \sum_{j=0}^6 \Delta_{ij} E_n^i W_R^j \quad (2.39)$$

Δ_{ij} = bivariant coefficients fit to the infinite media neutron spectra data

If, $W_R > 120.0 \text{ gm/cm}^2$,

$$f(W_R, E_n) = \left[\sum_{i=0}^4 \sum_{j=0}^6 \Delta_{ij} E_n^i 120.0^j \right] \exp \left[-\lambda(E_n) \cdot (W_R - 120.0) \right] \quad (2.40)$$

2.4.2 Gamma Ray

The KAP-V program calculates and prints as output data both the uncollided and collided (based on buildup factors) gamma ray functions.

The equation coded in the program for computing the response at the detector from each source point for each gamma ray energy group, E_G is:

$$D_\gamma(E_G) = \frac{\Gamma(E_G) \cdot S(\bar{R}_i, \bar{Z}_i, \bar{\theta}_k) \cdot \psi(\rho_m, E_G)}{4\pi \rho^2} \quad (2.41)$$

where: $\Gamma(E_G)$ = the source strength for each group, E_G , and all other terms were defined previously, except $\psi(\rho_m, E_G)$ which is defined below.

Uncollided Gamma Ray Flux

$$\psi(\rho_m, E_G) = \exp \left[-b_T(E_G) \right] \quad (2.42)$$

where: $b_T(E_G) = \sum_{m=1}^M \mu_m(E_G) \rho_m$ (2.43)

$\mu_m(E_G)$ = the gamma ray total absorption coefficient for each material, m , and each gamma ray group, E_G .

Collided Gamma Ray Flux

$$\psi(\rho_m, E_G) = B(\rho_m, E_G) \cdot \exp \left[-b_T(E_G) \right] \quad (2.44)$$

where: $B(\rho_m, E_G) = B[b_T(E_G)]$ (2.45)

or, if $b_T(E_G) > 20.0$,

$$B(\rho_m, E_G) = B(20.0)$$

and, $B[b_T(E_G)] = \sum_{i=0}^3 \beta_i(E_G) \cdot [b_T(E_G)]^i$ (2.46)

The quantities, β_i , are the coefficients for the cubic polynomial fit to the infinite media buildup factor data. These values of β_i can be either input to the code or obtained from the built-in library described in Section 2.6. In the above equation, $b_T(E_G)$, is not allowed to exceed 20.0 mean free paths, because the polynomial functions are not valid beyond this range.

The user will note that in equation 2.43, the units of $\mu_m(E_G)$ can be input as either cm^{-1} or cm^2/gm depending upon the units of ρ_m . If the library of the gamma ray absorption coefficients (see Section 2.6) is employed in the program, the units of $\mu_m(E_G)$ are cm^2/gm .

2.5 CONVERSION OF RADIATION LEVELS TO VARIOUS UNITS

A desirable feature of the KAP-V program is the ability to apply conversion factors to the total energy independent fast neutron response, the differential neutron energy response, and the gamma ray response. For example, application of the conversion factors to the gamma ray response could, at the option of the user, provide gamma ray output data in units of: Mev/ $\text{cm}^2\text{-sec}$, R/hr, R/hr-watt, Rads-carbon/hr, watts/gm-steel, watts/gm-aluminum, etc., all in one run on the computer.

First, the code calculates the total response at a given detector from all source points in a specific source region as follows:

$$DT_n = \sum_{\text{over all source points}} D_n = \text{Total energy-independent fast neutron response} \quad (2.47)$$

$$DT_n(E_n) = \sum_{\text{over all source points}} D_n(E_n) = \text{Total differential fast neutron energy response} \quad (2.48)$$

$$DT_\gamma(E_G) = \sum_{\text{over all source points}} D_\gamma(E_G) = \text{Total gamma ray response} \quad (2.49)$$

where: D_n' , $D_n(E_n)$, and $D_\gamma(E_G)$ were previously defined for each source point, in Equations 2.28, 2.34, and 2.41, respectively.

A set of ten energy independent conversion factors may be input to convert the quantity, DT_n' , to other units; a second set of ten conversion factors for each group, E_n , may be input to convert the quantity $DT_n(E_n)$, to other units; finally, a third set of ten conversion factors for each group, E_G , may be input to convert the quantity, $DT_\gamma(E_G)$, to other units. Therefore, for each detector point calculation, a total of thirty different responses may be obtained as output data.

The equations solved in applying the conversion factors are as follows:

Albert-Welton Function

$$\begin{aligned} DT'_{n,1} &= DT_n \cdot C_1 & (2.50) \\ DT'_{n,2} &= DT_n \cdot C_2 \\ &\vdots \\ &\vdots \\ &\vdots \\ \text{etc.} \end{aligned}$$

where: C_1, C_2, \dots, C_{10} = the input conversion factors, RSPA.

Differential Neutron Spectra Function

$$\begin{aligned} DT'_{n,1}(E_n) &= DT_n(E_n) \cdot C_1(E_n) & (2.51) \\ DT'_{n,2}(E_n) &= DT_n(E_n) \cdot C_2(E_n) \\ &\vdots \\ &\vdots \\ &\vdots \\ \text{etc.} \end{aligned}$$

where: $C_1(E_n), C_2(E_n), \dots, C_{10}(E_n)$ = each set of input conversion factors (RSPN), for each group, E_n .

Gamma Ray Function

$$DT'_{\gamma,1}(E_G) = DT_{\gamma}(E_G) \cdot C_1(E_G) \quad (2.52)$$

$$DT'_{\gamma,2}(E_G) = DT_{\gamma}(E_G) \cdot C_2(E_G)$$

•

•

•

etc.

where: $C_1(E_G), C_2(E_G), \dots, C_{10}(E_G)$ = each set of input conversion factors (RSPG), for each group, E_G .

The user will note that at least one set of response functions must be input for each function, DT_n , $DT_n(E_n)$, and $DT_{\gamma}(E_G)$. Otherwise the program will multiply by zero, and all values of DT'_n , $DT'_n(E_n)$, and $DT'_{\gamma}(E_G)$ will be printed as zeros.

2.6 LIBRARY DESCRIPTION

Gamma ray library data are included in the program to reduce tedious preparation of input data and to provide the user with latitude in specifying source gamma ray energies (e.g., a single 2.23 Mev gamma ray from hydrogen radiative capture or any of the radioisotope gamma ray source energies). The gamma ray library data consists of: 1) gamma ray mass absorption coefficients as a function of element atomic number (Z) and gamma ray energy (E_G), and 2) gamma ray buildup data as a function of material (H_2O , Al, etc.), type (energy, dose, energy absorption), and gamma ray energy (E_G). The calculated data requested from the library are stored internally by the program in the correct input data array.

2.6.1 Gamma Ray Mass Absorption Coefficient Data

Gamma ray absorption coefficients for each element can either be input to the program, or calculated from library data, or both. The program assumes the library calculated data to be the first set in the coefficient data, and the user (if using both options simultaneously) must input coefficient data at the proper addresses (see Section 4.4).

The gamma ray absorption coefficients in the library are calculated by two methods, depending on the atomic number of the element which is input to the program. These mass absorption coefficients are computed in units of cm^2/gm .

Method 1 ($1 \leq Z \leq 19$)

If the atomic number (Z) of the element is less than $Z = 20$, the program uses parabolic interpolation of tabulated data to obtain the mass absorption coefficient, $\mu/\rho(E_G)$, at energy E_G for the specified atomic number. A total of 19 sets of tabulated mass absorption coefficient data are included for $Z = 1$ through 19. Each set contains the 28 energy points presented in Reference 5. The 19 sets of element coefficient data were obtained from References 5 and 8.

Method 2 ($19 < Z \leq 92$)

If the atomic number (Z) of the element is greater than 19, an equation for $\left(\frac{\mu}{\rho}(E_G)\right)$ in the form of a bivariate polynomial is solved to obtain the mass absorption coefficient as a function of element atomic number (Z) and gamma ray energy (E_G). The mass absorption coefficient calculation follows as:

$$\left[\frac{\mu}{\rho}(E_G) \right]_m = \sum_{n=0}^3 \sum_{g=0}^3 \tau_{ng} (Z_m)^n \left(\frac{1}{E_G} \right)^g \quad (2.53)$$

The τ_{ng} 's are bivariate polynomial coefficients from Reference 5 fitted over two energy ranges of 0.2 to 2.0 Mev, and 2.0 to 10 Mev. If the input gamma ray energy is below 0.2 Mev, or greater than 10.0 Mev, the program sets the energy equal to the limit (0.2 or 10.0 Mev) and obtains the absorption coefficient at the energy limit.

2.6.2 Gamma Ray Buildup Factor Data

Gamma ray buildup factor data in the KAP V program can either be input to the program as specific energy group data, or the user may calculate the buildup factor using a bivariate polynomial equation. The program contains a library of a single set of bivariate polynomial data (Reference 6) that can be evaluated for each gamma ray energy group. Buildup factor input data to the program is restricted to the cubic polynomial data in the variable, $b_T(E_G)$. The buildup of gamma ray energy is represented by a single material. Hence, the user must select a set of polynomial data which is representative of the system material composition (e.g., in the NERVA-type reactor system, water buildup data is the most applicable data available).

The polynomial form of the buildup factor in the program is:

$$B(b_T(E_G)) = \sum_{i=0}^3 \beta_i(E_G) [b_T(E_G)]^i \quad (2.54)$$

If the user specifies the buildup factor library data, the control word IBILD and the input gamma ray energies, E_G , are used in computing the polynomial coefficients, $\beta_i(E_G)$, to be used in all buildup factor evaluations. The values of $\beta_i(E_G)$ are internally computed by the program from the bivariate polynomial expressions presented in Reference 6. These bivariate polynomials have certain restrictions which the program handles as follows:

1. If the gamma ray energy is less than or greater than the applicable energy range of the bivariate polynomial data, the buildup factor coefficients $\beta_i(E_G)$'s are evaluated at the lower or upper limit of the range (see Table 2).
2. If the polynomial's applicable range (TMFP) of depth penetration, $b_T(E_G)$, in mean free paths, is different than 20.0, the program assigns a value consistent with the buildup data to be used.
3. If the bivariate polynomial data is available for two ranges of energy, the program automatically selects the correct set of data to be evaluated for each energy.

TABLE 2
LIBRARY BIVARIANT POLYNOMIAL DATA
FOR GAMMA RAY BUILDUP COEFFICIENT EVALUATION

<u>IBILD Value</u>	<u>Material</u>	<u>Buildup Type</u>	<u>Applicable Ranges</u>		
			E_G (lower)	E_G (upper)	b_t (upper)*
1	Water	Dose	0.255	10.0	20.0
2	Water	Energy	"	"	"
3	Water	Energy Absorption	"		
4	Aluminum	Dose	0.5		
5	Aluminum	Energy			
6	Aluminum	Energy Absorption			
7	Iron	Dose			
8	Iron	Energy			
9	Iron	Energy Absorption			
10	Uranium	Dose			15.0
11	Uranium	Energy			
12**	Uranium	Energy Absorption			
13	Lead	Dose			
14	Lead	Energy			
15***	Lead	Energy Absorption			
17***	Tin	Dose			
19***	Tin	Energy			
21***	Tin	Energy Absorption			
23***	Tungsten	Dose			
25***	Tungsten	Energy			
27***	Tungsten	Energy Absorption	0.5	10.0	15.0

* b_t (lower) = 0.0 for all data, where b_t is the mean free path

**Data does not exist in Library

***Note: Odd numbered values are applicable between 0.5 and 4 Mev. Even numbered values, which are not given, are applicable between 4.0 and 10.0 Mev. The code automatically handles this problem.

The buildup factor library data presently available in the KAP V program is shown in Table 2 along with the applicable ranges of energy and depth penetration values which are built into the program. The input values of IBILD that will select the desired data are also presented.

The bivariate polynomials solved by the program to obtain the buildup factor coefficients, $\beta_i(E_G)$, are:

$$\beta_i(E_G) = \sum_{j=0}^J \xi_{ij}(Z_m)^j \quad (2.55)$$

or

$$\beta_i(E_G) = \sum_{j=0}^J \xi_{ij}\left(\frac{1}{Z_m}\right)^j \quad (2.56)$$

The program automatically selects the proper form of the polynomial as indicated by the input values of IBILD, E_G , and Z_m .

2.7 CALCULATIONS FOR A DETECTOR IN A SOURCE REGION

A basic difficulty with the point kernel technique is the calculation of the detector response when the detector is located in, or very close to, the source region. This difficulty arises when the denominator in the kernel approaches zero, i. e., in the equation, $\psi / 4\pi \rho^2$, if ρ (the distance from the source point to the detector point) is a very small number, the function, ψ , approaches infinity, and yields misleading, if not erroneous results. To obtain a meaningful (or valid) detector response when the source-detector separation distance is small, the spacing of source points in the near vicinity of the detector must be handled with extreme caution. The preceding discussion applies even when the option (described in this section) is used.

To compute the analytical response at the detector within a source region using this option, the control word ISCP must be input as the composition number of the source region of interest, and SMFP, which is the total mean free path of the source region composition (ISCP), must be input.

This option, which is provided to the program user, should be applied judiciously. Note that the option applies only to a gamma ray calculation. Also, the option is not applicable when the source point and the detector point are at the same identical location.

The program then solves the following equation:

$$\mu_s(E_G) = \sum_{m=1}^M \left[\frac{\mu}{\rho}(E_G) \right]_m \cdot \theta_{m, \text{ISCP}} \quad (2.57)$$

where:

$\mu_s(E_G)$ = the macroscopic gamma ray absorption coefficient (cm^{-1}) for the source region of interest for each energy group, G.

$\left[\frac{\mu}{\rho}(E_G) \right]_m$ = the microscopic gamma ray absorption coefficient (cm^2/gm) for each energy group, G.

$\theta_{m, \text{ISCP}}$ = the density (gm/cm^3) of each material, m, in the composition, ISCP.

In the above equation, $\left[\frac{\mu}{\rho} (E_G) \right]_m$, can be input as macroscopic data (cm^{-1}), if θ_m , ISCP are input as dimensionless volume fractions.

Next, the dimensionless quantity, ν , is calculated for each source point as:

$$\nu = \mu_s (E_G) \cdot \rho$$

where ρ is the total distance (cm) from a particular source point to the detector point, and $\mu_s (E_G)$ is defined above.

For each source-detector path calculation, ν is tested against the input quantity, SMFP. If, $\nu \geq \text{SMFP}$, the code calculates the usual attenuation function. But, if $\nu < \text{SMFP}$, the program solves the following equation as the gamma ray attenuation function, $D_\gamma (E_G)$, at the detector for each group, G :

$$D_\gamma (E_G) = \frac{B(E_G, \text{SMFP}) \cdot \bar{S}(E_G) \cdot (1 - \exp(-\text{SMFP}))}{\frac{4}{3} \pi \left(\frac{\text{SMFP}}{\mu_s(E_G)} \right)^3 \cdot (\mu_s(E_G))} \quad (2.58)$$

where:

$$\begin{aligned} B(E_G, \text{SMFP}) &= \text{the buildup factor for each group, } G, \text{ computed as: } \beta_0 + \beta, \text{ SMFP} \\ &= \beta_2(\text{SMFP})^2 + \beta_3(\text{SMFP})^3 \end{aligned}$$

$\bar{S}(E_G)$ = the source (e.g. watts or fissions per second) for the sphere whose radius is defined as ν for each group, G .

$\frac{4}{3} \pi \frac{\text{SMFP}}{\mu_s(E_G)}^3$ = the volume, cm^3 , of the sphere, calculated for each group, G .

The quantity, $\bar{S}(E_G)$, is calculated as: $\sum_{S'} S'$ all source points within radius, ν , where S'

is the source for each volume in which a source point is located. Equation 2.58 is simply the solution of the response at a detector point located at the center of a spherical source of ν mean free paths in radius as given in reference 9, page 371.

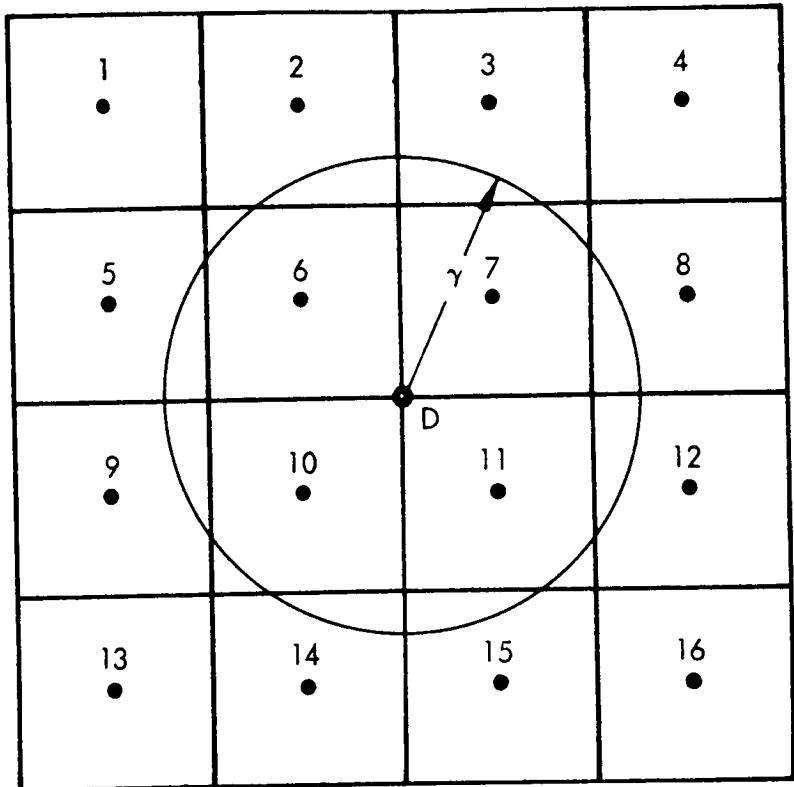
The quantity, $D_\gamma(E_G)$, is added to the detector response calculated for all the source points external to the sphere. Two examples are given to illustrate some problems associated with the option.:

1. Figure 4 shows an r, z plane cut through an "exclusion" sphere of radius, v . A typical source point mesh is also shown in Figure 4. The detector point is located at D, the center. For this example, attenuation functions for source points, 1-4, 5, 8, 9, and 12-16 would be calculated by the usual KAP V equations. The source strengths for the volume elements surrounding points 6, 7, 10, and 11 would be used in the "sphere" option.

One can observe that a part of the volume surrounding points 6, 7, 10, and 11 is outside the sphere, but are included in the total source calculations for the sphere; and, that part of the element volume surrounding point no. 8 (for example) which actually lies within the sphere, is not included in the total source for the sphere.

These approximations are part of the option, and introduce some uncertainties in the answer.

2. Figure 5 shows a sphere overlapping two regions having different composition numbers. The zone with composition no. 1 is not a source region. But that portion of the sphere which extends into the region (shown by the shaded area) does, in fact, have a source strength associated with it. Hence, a calculation using this option (as coded) when the detector is near the boundary of a source region, introduces uncertainties in the answer.



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Figure 4. Example of "Exclusion" Sphere Option

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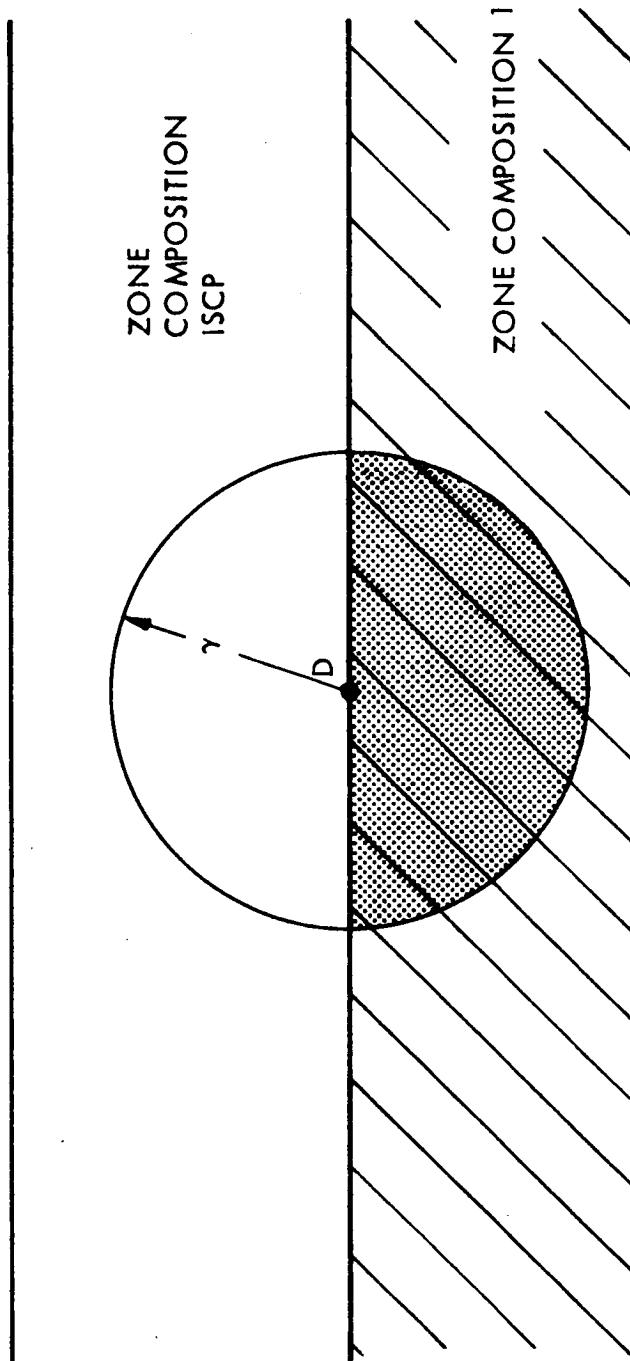
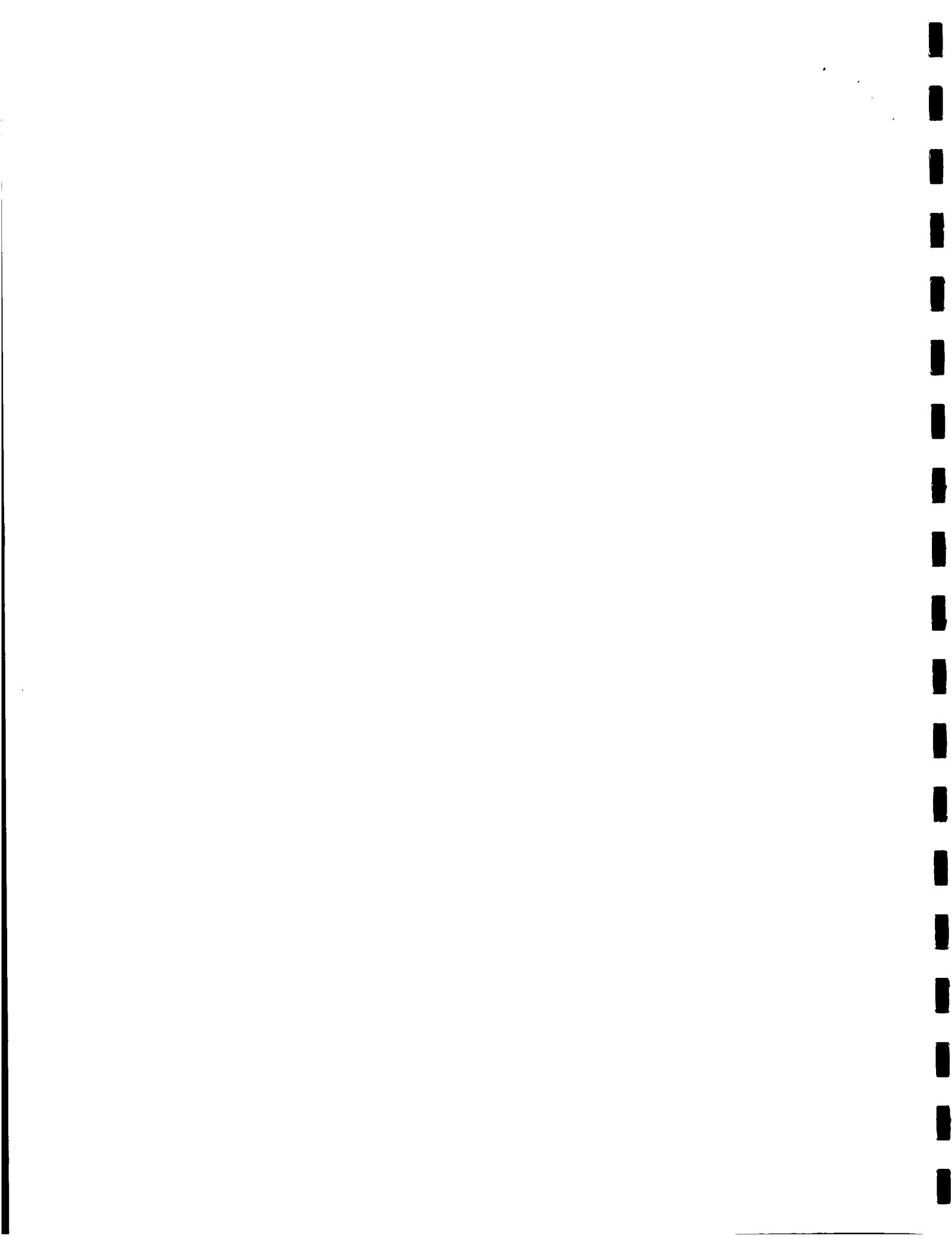


Figure 5. Example of "Exclusion" Sphere Option



SECTION

3.0 PROGRAM LOGIC

The program logic for the KAP-V program is presented in this section. The flow chart is constructed in a simplified form to illustrate to the user when a certain operation or calculation is performed. The symbolism used in the flow chart is shown in Figure 6 and the actual KAP-V logical flow chart is given in Figure 7. Each of the principal operations performed by KAP-V are described in previous sections. The logic of the program as presented in Figure 7 has the principal FORTRAN DO loops indicated as indexing loops A-D for simplicity.

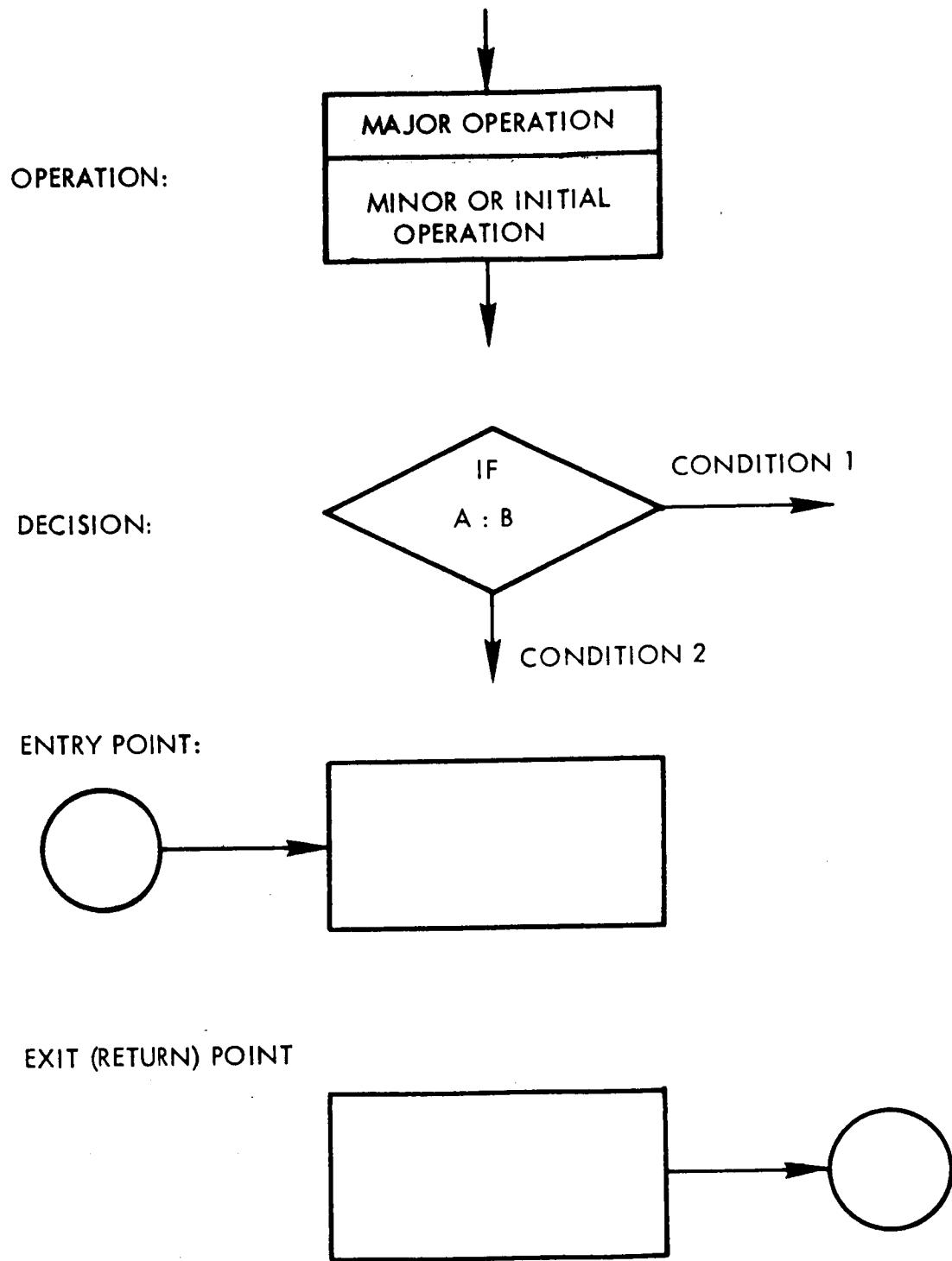


Figure 6. Flow Chart Symbolism

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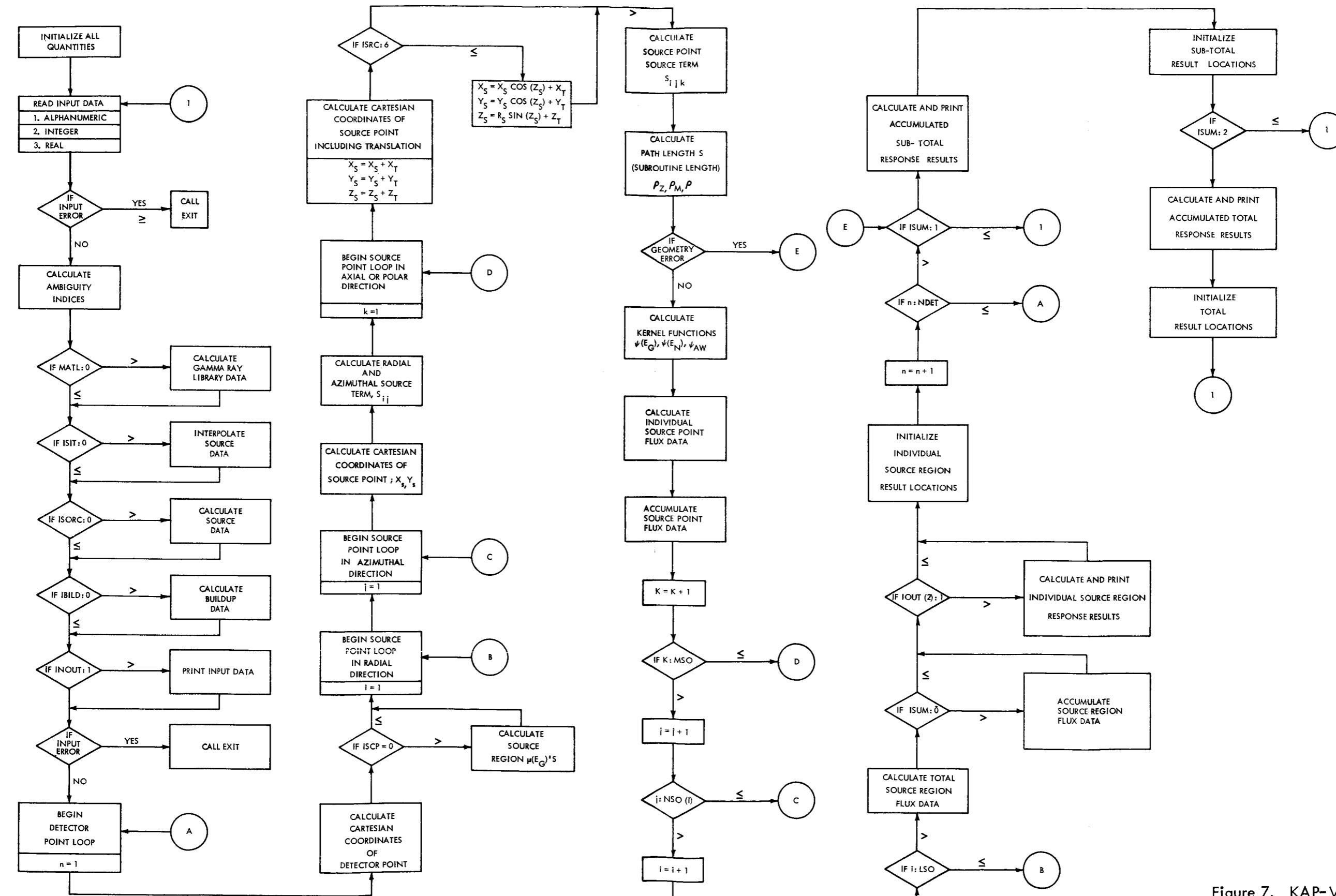


Figure 7. KAP-V Program Logic

SECTION

4.0 INPUT DATA INSTRUCTIONS

4.1 GENERAL DESCRIPTION

The input data to each KAP-V problem or "change case" consists of three types of input data. The types of data are:

- 1) Alphanumeric data
- 2) Integer or fixed point data
- 3) Real or floating point data

The general card format consists of three subfields (card columns 1 - 12) for all types of data, and a subdivided fourth field (card columns 13 - 72) for alphanumeric, integer, or real data. The first three subfields which are read in a (I2, I1, I9) FORTRAN format require the following information which are common to each type of data input:

- 1) The number of pieces or words of data on the card (right adjusted*).
- 2) The last card of a particular type of data (i.e., 0 or blank means that more cards of a particular type follow; 1 means this is the last card of a particular type of data).
- 3) The address or data location of the first piece or word of data on the card. All subsequent data in the fields on the card, up to and including the total pieces of data specified in the first subfield, are stored in sequence from the card address.

The fourth subfield which is divided according to the type of data to be read is described in the following sections. It must be noted that each problem which is input to the program must include at least one data card of each type (alphanumeric, integer, and real). Therefore, the minimum card count for a stacked problem is three cards, one for each type of data with a 1 in column 3 of each card.

The ability to assign the specific address of each data word within any data array allows the user to run stacked problems with minor data changes with a minimum card count.

*Right adjusted means the least significant digit of the number is at the extreme right of the field.



4.2 ALPHANUMERIC DATA

The card format for the alphanumeric or title data is the FORTRAN format, (I2, I1, I9, 15A4). The last 15 data words (card columns 13 - 72) are subdivided into four character alphanumeric data which are input at the discretion of the user.

The relative location or address of the initial four character word in each title section is tabulated and described below and the breakdown of data sections are described. The option to input specific title data was included because of the output flexibility of the program. It must be noted that the user, in specifying a three word title, has the capability of inputing 12 characters of information. In addition, the breakdown of the 180 and 120 character titles into 3 and 2 lines of 15 alphanumeric data words per line (60 characters per line) must be noted by the user to provide clearly titled output results.

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
1	TITLE	(45)	Overall problem title (180 alphanumeric characters) which is output at the beginning of the output printing. (Printed 60 characters to a line.)
46	TITLE	(30)	Title information (120 characters) printed preceding the output of results for each source region and all detector points. (Printed 60 characters to a line.)
76	TITLE	(30)	Title information (120 characters) printed preceding the output of results for the subtotal over a selected set of source regions (i. e., the summation overall reactor subregions). (Printed 60 characters to a line.)
106	TITLE	(30)	Title information (120 characters) printed preceding the output of results for the summation of all source regions, (i. e., summation over subtotals). (Printed 60 characters to a line.)
	TITLE	(3, 10)	Title information (3 words or 12 characters) associated with gamma ray response output data.

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
136	TITLE (1)		Gamma ray response No. 1
139	TITLE (2)		Gamma ray response No. 2
142	TITLE (3)		Gamma ray response No. 3
145	TITLE (4)		Gamma ray response No. 4
148	TITLE (5)		Gamma ray response No. 5
151	TITLE (6)		Gamma ray response No. 6
154	TITLE (7)		Gamma ray response No. 7
157	TITLE (8)		Gamma ray response No. 8
160	TITLE (9)		Gamma ray response No. 9
163	TITLE (10)		Gamma ray response No. 10
166	TITLE	(3, 10)	Title information (3 words or 12 characters) associated with each neutron response output data. Data order identical to gamma ray response data.
196	TITLE	(3, 10)	Title information (3 words or 12 characters) associated with each Albert-Welton response output data. Data order identical to gamma ray response title data.
	TITLE	(3, 25)	Title information (3 words or 12 characters) for each detector point in the problem:
226	TITLE (1)		Detector point 1
229	TITLE (2)		Detector point 2
232	TITLE (3)		Detector point 3
235	TITLE (4)		Detector point 4
•	•		•
291-300	TITLE (25)		Detector point 25

4.3 INTEGER OR FIXED POINT DATA

The card format for the integer data is the FORTRAN format (I2, I1, I9, 20I3). The last 20 pieces of data (card columns 13 - 72) are subdivided into three digit fields which are input as "right adjusted" integer data.



The address of each piece of data, or the address of each data array, is tabulated and described below. The addressing of data internal to a data array is also tabulated and described.

Integer or Fixed Point Data

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
1	NGG		Total number of gamma ray groups, E_G . ($1 \leq NGG \leq 30$)
2	NGN		Total number of neutron groups, E_n . ($1 \leq NGN \leq 30$)
3	MAT		Total number of materials or elements (i. e., an element, H, O, or Fe; or a material, H_2O , UO_2) in the material/composition table. Note: The user may input the data, $\mu_m(E_G)$ for MAT elements, or optionally the program will calculate gamma ray coefficients, $\mu_m(E_G)$, for MATL elements, but not materials (such as H_2O). The coefficients, $\mu_m(E_G)$, calculated by the program are in units of cm^2/gm . The internally generated gamma ray data will appear as the first MATL sets of data in the material/composition table. Therefore, the MAT set of data must correspond up to and including the first MATL set of data. ($1 \leq MAT \leq 20$) and ($MAT \leq MATL$)
4	NCOMP		Total number of compositions in the material/composition table. ($1 \leq NCOMP \leq 50$)
5	NDET		Total number of detector points to be evaluated for the source region in the problem. The program will accumulate results for multiple source regions for all detector points under the control of the input quantity, ISUM. ($1 \leq NDET \leq 25$)

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
6	NBOUND		Total number of boundary surfaces in the problem geometry. $(1 \leq NBOUND \leq 100)$
7	NREG		Total number of geometric regions or zones in the problem geometry. A region is described by 6 or less boundary surfaces which subdivide the overall problem space. $(1 \leq NREG \leq 100)$
8	NRSPG		Total number of sets of response functions to be applied to the calculated gamma ray flux data at each detector point. $(1 \leq NRSPG \leq 10)$
9	NRSPN		Total number of sets of response functions to be applied to the calculated neutron flux data at each detector point. $(1 \leq NRSPN \leq 10)$
10	NRSPA		Total number of response functions to be applied to the Albert-Welton neutron dose function results at each detector point. $(1 \leq NRSPA \leq 10)$
11	MATL		Total number of elements for which gamma ray coefficient sets are to be internally generated by the program. The MATL sets must be the first MAT sets of the material/composition table. $(0 \leq MATL \leq MAT)$
19	IBILD		Control word for buildup factor input data. IBILD = 0: input buildup factor polynomial coefficients at location BILD IBILD > 0: the program internally computes the buildup factor coefficients from the library (see Section 2.6) If IBILD = 1, H ₂ O dose buildup If IBILD = 2, H ₂ O energy buildup If IBILD = 3, H ₂ O energy absorption buildup

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
			If IBILD = 4, A1 dose buildup
			If IBILD = 5, A1 energy buildup
			If IBILD = 6, A1 energy absorption buildup
			If IBILD = 7, Fe dose buildup
			If IBILD = 8, Fe energy buildup
			If IBILD = 9, Fe energy absorption buildup
			If IBILD = 10, U dose buildup
			If IBILD = 11, U energy buildup
			If IBILD = 12, U energy absorption buildup
			If IBILD = 13, Pb dose buildup
			If IBILD = 14, Pb energy absorption buildup
			If IBILD = 15, Pb energy absorption buildup
			If IBILD = 17, Sn dose buildup
			If IBILD = 19, Sn energy buildup
			If IBILD = 21, Sn energy absorption buildup
			If IBILD = 23, W dose buildup
			If IBILD = 25, W energy buildup
20	IGAM		Control word for calculation of gamma ray attenuation functions. IGAM = 0: Do not calculate gamma ray attenuation functions. IGAM = 1: Calculate gamma ray attenuation functions.
	INEUT	(3)	Control words for calculation of neutron attenuation functions. INEUT (i) = 0: Do not calculate neutron attenuation function. INEUT (i) = 1: Calculate neutron attenuation function.

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
21	INEUT(1)		Control word for Albert-Welton neutron dose calculation.
22	INEUT(2)		Control word for monovariant polynomial, $f(W_R, E_n)$, neutron spectra calculation.
23	INEUT(3)		Control word for bivariate polynomial, $f(W_R, E_n)$, neutron spectra calculation.
24	ISCP		Control word for calculation at a detector in the immediate vicinity of source points. (An analytical result is calculated if the path length between a source point and detector point is less than or equal to SMFP mean free paths in the ISCP composition. (See Section 2.7.) ISCP = 0: Do not calculate analytic result. ISCP > 0: Calculate analytic result with the ISCP composition as the source material.
25	IZSO		The number of the source zone in which all path length calculations are initiated. IZSO = Source zone number.
26	ISORC		Control word for calculation of all source distribution functions. ISORC = 0: Do not calculate source distribution data but use previous problem data. ISORC = 1: Calculate new source distribution data from RS, ZS, PHI, and FSI input data.
27	ISRC		Control word for the calculation of radial source distribution data. ISRC = 0: Do not calculate and renormalize input data, but use RS, ZS, PHI, and FSI as point source data. (This option allows the description of one or more discrete point sources.)

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
			<p><u>Note:</u> The following radial source distribution options are applicable to a cylindrical source geometry. (See Section 2.3.2.)</p> <p>ISRC = 1: Uniform or flat source distribution (Does not require FSI input).</p> <p>ISRC = 2: Cosine source distribution based on input data, XI's. (Does not require FSI input.)</p> <p>ISRC = 3: Source distribution based on a linear variation of input data, FSI, between mesh points, RS.</p> <p>ISRC = 4: Source distribution based on input exponential distribution data XI's. (Does not require FSI input.)</p> <p>ISRC = 5: Source distribution based on exponential variation of input data, FSI, between mesh points, RS.</p>
			<p><u>Note:</u> The following radial source distribution options are applicable to a spherical source geometry.</p> <p>ISRC = 6: Uniform or flat source distribution. (Does not require FSI input data.)</p> <p>ISRC = 7: Source distribution based on a linear variation of input data, FSI, between mesh points, RS.</p>
28	ISZC		<p>Control word for axial or polar source distribution calculations.</p> <p><u>Note:</u> The following source distribution options are applicable to cylindrical source geometry. (See Section 2.3.2.)</p> <p>ISZC = 1: Uniform or flat source distribution. (Does not require FSI input.)</p> <p>ISZC = 2: Cosine source distribution based on the input data, ETA, (Does not require FSI input data.)</p> <p>ISZC = 3: Source distribution based on the linear variation of input data, FSI, between the mesh points, ZS.</p> <p>ISZC = 4: Source distribution based on an exponential variation of the source between mesh points, ZS, and the input data, ETA. (Does not require FSI input data.)</p>

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
29	ISTC		<p>ISZC = 5: Source distribution based on the exponential variation of the source between mesh points, ZS. (Requires FSI input data.)</p> <p><u>Note:</u> The following source distribution option is applicable to a spherical source geometry.</p> <p>ISZC = 6: Uniform or flat polar variation. (Does not require FSI input data.)</p> <p>Control word for azimuthal source description. (See Section 2.3.2.)</p> <p><u>Note:</u> All azimuthal source distributions are assumed to be uniform. The user must use the ISRC = 0 option with all input data calculated externally to the program to do otherwise.</p> <p>ISTC = 1: Azimuthal source point spacing from the input data, PHI, with NSO (1) intervals in each radial interval.</p> <p>ISTC = 2: Azimuthal source point spacing from the input data, PHI, with the number of azimuthal intervals for each radial interval set by the input data, NSO.</p> <p>ISTC = 3: Azimuthal source point spacing based on equal azimuthal intervals; the number of intervals is set by the input data, NSO.</p>
30	ISIT		<p>Control word for source distribution interpolation calculation.</p> <p>ISIT = 0: Do not interpolate RSIT, ZSIT, FSIT input data to obtain FSI at RS and ZS.</p> <p>ISIT = 1: Interpolate RSIT, ZSIT, FSIT input data to obtain FSI at RS and ZS.</p>
31	ISUM		<p>Control word for summation of the contribution of individual source regions to the detector response at each detector point.</p>

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
			ISUM = 0: Do not include this region in the summation of individual source region results.
			ISUM = -1: Start summing individual source region results beginning with this region.
			When ISUM = -1: The storage allocated for the grand total and subtotal is set to zero. The results for the region are added to the grand total and subtotal. Then ISUM is set equal to +1.
			ISUM = +1: Add individual source region results to the grand total and the subtotal.
			ISUM = 2: Add the contribution of the source region to the subtotal and grand total, print the subtotal, then set the subtotal to zero.
			ISUM = 3: Add the contribution of the source region to the subtotal and the grand total, print the subtotal and the grand total, set the grand total and subtotal to zero.
32	IOUT(1)		<p>Control word for printing <u>input</u> data.</p> <p>IOUT(1) = 0: Do not print input data.</p> <p>IOUT(1) = 1: Print card images of input data.</p> <p>IOUT(1) = 2: Print card images and organized input data.</p> <p><u>Note:</u> Normalized source distribution data are printed only when IOUT(1) = 2.</p>
33	IOUT(2)		<p>Control word for printing of <u>output</u> data.</p> <p>IOUT(2) = 1: Do not print individual source region results.</p> <p>IOUT(2) = 0: Print all output data for each individual source region.</p>

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
	KORD	(3)	Degree or order for each set of the neutron moments method bivariate polynomial data in the independent variable, energy (E_n).
35	KORD(1)		Degree or order of the first set of coefficients for the energy range, BKP(1) to BKP(2).
36	KORD(2)		Degree or order of the second set of coefficients for the energy range, BKP(2) to BKP(3).
37	KORD(3)		Degree or order of the third set of coefficients for the energy range, BKP(3) to BKP(4).
	IORD	(3)	Degree or order for each set of the neutron moments method bivariate polynomial data in the independent variable, depth penetration (W_R).
40	IORD(1)		Degree or order of the first set of coefficients for the energy range, BKP(1) to BKP(2).
41	IORD(2)		Degree or order of the second set of coefficients for the energy range, BKP(2) to BKP(3).
42	IORD(3)		Degree or order of the third set of coefficients for the energy range, BKP(3) to BKP(4).
50	LSO		Total number of radial mesh intervals in the source region description. $(1 \leq LSO \leq 20)$
51	MSO		Total number of axial or polar mesh intervals in the source region description. $(1 \leq MSO \leq 20)$
	NSO	(20)	Total number of azimuthal mesh intervals for each radial interval in the source region description. $(1 \leq NSO(i) \leq 20)$

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
52	NSO(1)		Total number of azimuthal mesh intervals in radial interval number 1 or if, ISTC = 1, the total number of intervals in each radial interval.
53	NSO(2)		Total number of azimuthal mesh intervals in radial interval 2.
•	•		•
•	•		•
•	•		•
71	NSO(20)		Total number of azimuthal mesh intervals in radial interval 20.
75	LSIT		Total number of radial source distribution input data values, RSIT and FSIT, to be used in interpolation of source distribution data, FSI, at the mesh points RS. ($1 \leq LSIT \leq 21$)
76	MSIT		Total number of axial source distribution input data values, ZSIT and FSIT, to be used in interpolation of source distribution data, FSI, at the mesh points, ZS. ($1 \leq MSIT \leq 21$)
	NEQBD(j)	(100)	Surface equation type number for each surface or boundary, j, in the problem. (The surface equations and their respective type number are presented preceding the surface equation coefficient input description). ($1 \leq NEQBD(j) \leq 6$)
100	NEQBD(1)		Type number for surface No. 1.
101	NEQBD(2)		Type number for surface No. 2.
•	•		•
•	•		•
•	•		•

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
199	NEQBD(100)		Type number for surface No. 100.
	NBNDZN(i) (100)		The total number of boundaries or surfaces defining each zone, i, in the problem. Each zone must have at least one boundary and no more than six boundaries. $(1 \leq NBNDZN(i) \leq 6)$.
			<u>Note:</u> The sign (+) of NBNDZN(i) denotes whether the zone is an outside or last zone and the user must specify this sign. If NBNDZN is negative, the zone is an outside or last zone.
200	NBNDZN(1)		Total number of boundary surfaces for zone No. 1
201	NBNDZN(2)		Total number of boundary surfaces for zone No. 2.
•	•		•
•	•		•
•	•		•
299	NBNDZN(100)		Total number of boundary surfaces for zone No. 100.
	NCMPZN(i) (100)		The composition number of the mixture of materials in zone, i, for each zone in the problem.
300	NCMPZN(1)		Composition number in zone 1.
301	NCMPZN(2)		Composition number in zone 2.
•	•		•
•	•		•
•	•		•
399	NCMPZN(100)		Composition number in zone 100.
	LBD(j, i) (6, 100)		The boundary surface numbers, j, for each zone, i. The user must specify NBNDZN(i) surface numbers for each zone, i, and the surfaces must totally enclose the zone or region in the problem. Outside zones can be described as single boundary zones.
			<u>Note:</u> The KAP-V program will automatically assign all ambiguity indices (+ or -) of each surface, j, in relation to each zone, i.



<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
500	LBD(j, 1)		Surface numbers, $j = 1, NBNDZN(1)$, for zone 1.
506	LBD(j, 2)		Surface numbers, $j = 1, NBNDZN(1)$, for zone 2.
•	•		•
•	•		•
•	•		•
1094	LBD(j, 100)		Surface numbers, $j = 1, NBNDZN(1)$, for zone 100.
	NTRYZN(j, i) (6,100)		The zone identification number for each boundary, j , of each zone, i , which defines the zone encountered upon crossing each boundary of the zone, i . There is a one to one correspondence between LBD and NTRYZN. Note: If more than one zone can be entered upon crossing boundary, j , the user can minimize problem running times by specifying the zone entered the most times, or if this can not be determined, the zone with the lower identification number.
1100	NTRYZN(j, 1)		Zone number, $j = 1, NBNDZN(1)$, for zone 1.
1106	NTRYZN(j, 2)		Zone number, $j = 1, NBNDZN(1)$, for zone 2.
•	•		•
•	•		•
•	•		•
1694	NTRYZN(j, 100)		Zone number, $j = 1, NBNDZN(1)$, for zone 100.

4.4 REAL OR FLOATING POINT DATA

The card format for the real data is the FORTRAN format (12, 11, 19, 5E12.5). The last 5 pieces of data (card columns 13-72) are subdivided into twelve digit fields which are input as real or floating point data.

The relative location or address of each piece of data, or the initial address of each data array, is tabulated and described below. The addressing of data internal to a data array is also tabulated and described.

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
	ASOI	(2)	Gamma ray and neutron source normalization constants. The input constants, ASOI, must be dimensionally consistent with GSOUR, NSOUR, and AWSOUR to provide particles/cm ³ -sec.
1	ASOI(1)		Gamma ray source normalization constant.
2	ASOI(2)		Neutron source normalization constant. Note: If ASOI(2) is input as 0.0, the program assumes ASOI(2) = ASOI(1) and all gamma ray source distribution data, FSI, are also used for neutron source calculations.
	XI	(2)	Radial source distribution constants used in the truncated cosine or exponential source distribution function.
3	XI(1)		<u>For truncated cosine</u> , = π/R^* , where R* is the extrapolated region radius.
4	XI(2)		= 0.0
	XI	(2)	<u>For exponential</u> = $f(R_o)$, the source value at the left boundary radius, R_o
4	XI(2)		= σ , the slope of the source distribution in the region.
	ETA	(2)	Axial source distribution constants used in the truncated cosine or exponential source distribution function.
	ETA	(2)	<u>For truncated cosine</u> = $\frac{\pi}{H^*}$, where H* is the extrapolated height.
5	ETA(1)		= $\frac{H^*}{2}$
	ETA	(2)	<u>For exponential</u> = $f(Z_o)$, the source value at the left axial boundary, Z_o .
6	ETA(2)		= σ , the slope of the source distribution in the region.
7	RS(i)	(21)	Radial dimensions of the source region mesh intervals. (LSO + 1 values)



<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
28	ZS(m)	(21)	Axial or polar dimensions of source region mesh intervals. (MSO + 1 values)
	PHI(n, i)	(21,20)	Azimuthal dimensions of source region mesh intervals for each radial interval, i.
49	PHI(n, 1)		Dimensions of radial interval No. 1, (NSO(1) + 1 values)
70	PHI(n, 2)		Dimensions of radial interval No. 2, (NSO(2) + 1 values)
•	•		•
•	•		•
•	•		•
448	PHI(n, 20)		Dimensions of radial interval No. 20, (NSO(20) + 1 values) <u>Note:</u> PHI(n, i) for K greater than 1 are required only when for ISTC = 2
	FSI	(21, 22, 2)	Source distribution data for radial, axial, or polar, and azimuthal distribution for both gamma ray, K = 1, and neutron, K = 2, source data.
470	FSI(l, 1, 1)		Gamma ray radial source data, (LSO + 1 values)
491	FSI(m, 2, 1)		Gamma ray axial or polar source data, (MSO + 1 values)
512	FSI(n, 3, 1)		Gamma ray azimuthal data for each radial interval, (NSO(1) + 1 values) <u>Note:</u> FSI (n, 3, 1) is required input only for the case when the user specifies all source data FSI as input (i. e., a unit point source is input by specifying ISORC = 1, ISRC = 0, FSI(470) = 1.0, FSI(491) = 1.0, FSI(512) = 1.0)
512	FSI(1, 3, 1)		Gamma ray azimuthal data for radial interval No. 1.
533	FSI(2, 3, 1)		Gamma ray azimuthal data for radial interval No. 2.
•	•		•
•	•		•
•	•		•
931	FSI(20, 3, 1)		Gamma ray azimuthal data for radial interval No. 20.

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
932	FSI(1, 1, 2)		Neutron source distribution data input which is identical in order to the gamma ray data, but with all addresses increased by 462. Note: If ASOI(2) is input as 0.0, then FSI(1, 1, 2) = FSI(1, 1, 1)
1400	GSOUR	(30)	Gamma ray source by energy group, ENG. Note: GSOUR must be dimensionally consistent with ASOI(1), so that GSOUR(k)·ASOI(1) will provide the units of particles or Mev/cm ³ -sec.
1430	NSOUR	(30)	Neutron source by energy group, ENN. Note: NSOUR must be dimensionally consistent with ASOI(2). These quantities provide the user with the capability to input group dependent integration factors (energy band widths) and <u>must not be</u> construed as neutron source spectra.

The following input data (ABD, BBD, CBD, XOB_D, YOB_D, ZOB_D, DBD) are the surface equation coefficients and constants for each boundary (1-100). This input depends on the boundary or surface equation type. The surface equation types which are in the program are the general quadratic equation, and five of the common degenerate forms, as shown below:

<u>Surface Equation Type</u>	<u>Equation</u>
1	$AX^2 + XX_0 + BY^2 + YY_0 + CZ^2 + ZZ_0 = D$
2	$A(X - X_0)^2 + B(Y - Y_0)^2 + C(Z - Z_0)^2 = D$
3	$(X - X_0)^2 + (Y - Y_0)^2 = D$
4	$X = D$
5	$Y = D$
6	$Z = D$

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
1460	ABD	(100)	Surface equation coefficient constant, A, for surfaces 1-100.
1560	BBD	(100)	Surface equation coefficient, B, for surfaces 1-100.
1660	CBD	(100)	Surface equation coefficient, C, for surfaces 1-100.
1760	XOBD	(100)	Surface equation constant, X_0' , for surfaces 1-100.
1860	YOBD	(100)	Surface equation constant, Y_0' , for surfaces 1-100.
1960	ZOBD	(100)	Surface equation constant, Z_0' , for surfaces 1-100.
2060	DBD	(100)	Surface equation constant, D, for surfaces 1-100. <u>Note:</u> To eliminate errors in path length calculations, the program automatically squares the input quantity, D, for surface equation types 2 and 3 to provide absolute matching of surface intersections. Therefore, input the quantity, D, as the radius of interest.
XYZ(i, i)		(3,100)	Cartesian coordinates of the point internal to each zone, i, described by the input data, LBD. There are 3 x NREG required input values. These data are used in computing the ambiguity indices (+ or -) of each surface in relation to the zone and extreme caution must be used in determining input values.
2160	XYZ(i, 1)		(X_p', Y_p', Z_p') for zone 1.
2163	XYZ(i, 2)		(X_p', Y_p', Z_p') for zone 2.
•	•		•
•	•		•
•	•		•
2397	XYZ(i,100)		(X_p', Y_p', Z_p') for zone 100.
COMP(m, n)		(20,50)	Composition matrix (densities or volume fractions) according to the materials or elements, m, in the problem.

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
2460	COMP(m, 1)		COMP(m, 1): data for all materials, m, in composition 1.
2480	COMP(m, 2)		COMP(m, 2): data for all materials, m, in composition 2.
•	•		•
•	•		•
•	•		•
3440	COMP(m, 50)		COMP(m, 50): data for all materials, m, in composition 50.
3460	ENN	(30)	Representative energy of each neutron group.
	XSECN(m, i)	(20, 3)	Data for the Albert-Welton and neutron spectra functions.
3490	XSECN(m, 1)		Neutron removal cross sections for each material for use with the Albert-Welton function.
3510	XSECN(m, 2)		Constants (η 's) for each material for use with the Albert-Welton function.
3530	XSECN(m, 3)		Neutron removal cross sections for each material for use with the neutron spectra function.
3550	XSNREF		Neutron removal cross section for the material for which the neutron moments data is input (reference material removal cross section).
3551	ALFA	(7)	Constants (α 's) for the Albert-Welton function.
3558	AWSOUR		Source strength to be applied to the Albert-Welton kernel. <u>Note:</u> AWSOUR must be dimensionally consistent with ASOI(2).
3560	ENG	(30)	Representative energy of each gamma ray source group.
	XSECG(k, m)	(30, 20)	Gamma ray absorption coefficients for each group, k, and each material, m, in the problem. The program will generate these data from tables and/or polynomial evaluation if requested. (See Section 2.6.)
3590	XSECG(k, 1)		Coefficients for each group k, material 1.

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
3620	XSECG(k, 2)		Coefficients for each group k, material 2. • • • • • •
4160	XSECG(k, 30)		Coefficients for each group k, material 30.
	BILD(1, k)	(4, 30)	Gamma ray cubic polynomial build up coefficients for each gamma ray group, k. <u>Note:</u> The program will internally compute these data if so requested. (See Section 2.6.)
4190	BILD(1, 1)		$\beta_0, \beta_1, \beta_2, \beta_3$ for group 1.
4194	BILD(1, 2)		$\beta_0, \beta_1, \beta_2, \beta_3$ for group 2. • • • • • •
4306	BILD(1, 30)		$\beta_0, \beta_1, \beta_2, \beta_3$ for group 30.
4310	SMFP		Total mean free path of source material (source composition ISCP) used in determining the exclusion sphere volume for source points adjacent to detector points. This quantity is used in the empirical solution of the gamma ray flux for detector points internal to source regions. (See Section 2.7)
4311	TMFP		The limit or maximum range of mean free paths of gamma ray depth penetration for cubic polynomial buildup data. The program calculates buildup only on TMFP (or less) mean free paths. If the mean free path exceeds TMFP, the program sets the mean free path equal to TMFP. The program assumes TMFP = 20, if TMFP is not input.
4312	EPSLN		Surface equation-path length calculation error limit used in determining if a surface is crossed. If the test fails, an error statement is given. <u>Note:</u> EPSLN is internally set as 1.0×10^{-6} and is not required as input if the user accepts this value.

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
4313	FUDGE		<p>Surface equation-path length calculation step quantity used in providing a means for the calculation to cross a boundary. If two steps are unsuccessful, an error statement is given.</p> <p><u>Note:</u> FUDGE is internally set as 1.0×10^{-3} and is not required as input if the user accepts this value.</p>
4314	BKP(i)	(4)	<p><u>For Monovariant Neutron Spectra Data:</u></p> <p>The neutron depth penetration, gm/cm^2, which is the breakpoint between the two sets of monovariant moments method data. Only BKP(1) is required.</p> <p><u>For Bivariant Neutron Spectra Data:</u></p> <p>The neutron energy breakpoints for the applicability of the bivariant polynomial data. BKP(1-4) must be in order of decreasing energy <u>and</u> the last values 2, 3 or 4 must not be zero (e.g., for only one set of polynomial data, BKP(1) = E (higher) and BKP(2, 3, and 4) = E (lower)).</p>
	COM	(5, 30, 2)	<p>Neutron spectra monovariant polynomial coefficients for NGN groups. This data, which is evaluated as a function of depth penetration (W, gm/cm^2 of equivalent neutron attenuation), is assumed to be applicable in the range of $0.0 \leq W \leq 120.0 \text{ gm/cm}^2$. The two sets of input data divide this range into, $0.0 \leq W \leq \text{BKP}(1)$ and $\text{BKP}(1) \leq W \leq 120.0$. For any depth penetration in excess of 120.0 gm/cm^2, the group dependent λ's (input quantities, XLAM) are used as simple exponential attenuation as, $\exp[-\lambda(W - 120.0)]$.</p>
4320	COM(i, 1, 1)		COM(1-5): C_5, C_4, C_3, C_2, C_1 for group 1, and $W \leq \text{BKP}(1)$.
4325	COM(i, 2, 1)		COM(1-5): C_5, C_4, C_3, C_2, C_1 for group 2, and $W \leq \text{BKP}(1)$.
•	•		•
•	•		•
•	•		•
4460	COM(i, 30, 1)		COM(1-5): C_5, C_4, C_3, C_2, C_1 for group 30, and $W \leq \text{BKP}(1)$.

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
4470	COM(i, 1, 2)		COM(1-5): C_5, C_4, C_3, C_2, C_1 for group 1, and $W \geq BKP(1)$.
4475	COM(i, 2, 2)		COM(1-5): C_5, C_4, C_3, C_2, C_1 for group 2, and $W \geq BKP(1)$.
•	•		•
•	•		•
•	•		•
4615	COM(i, 30, 2)		COM(1-5): C_5, C_4, C_3, C_2, C_1 for group 30, and $W \geq BKP(1)$.
	CON	(5, 5, 4)	Neutron spectra bivariate polynomial coefficients. These data, which are evaluated as a function of depth penetration, W , and neutron energy, E_n , is assumed applicable over the entire range of $W \geq 120.0 \text{ gm/cm}^2$. The four sets of data divide the energy range into four intervals as determined by $BKP(1-4)$. Calculations for W in excess of 120.0 gm/cm^2 are discussed above in the monovariate polynomial description.
4620	CON(5, 5, 1)		CON(1-25): $C_1, C_2, C_3, C_4, C_5 \dots C_{25}$ for $BKP1 \geq E \geq BKP2$.
4645	CON(5, 5, 2)		CON(1-25): $C_1, C_2, C_3 \dots C_{25}$ for $BKP2 \geq E \geq BKP3$.
4670	CON(5, 5, 3)		CON(1-25): $C_1, C_2, C_3 \dots C_{25}$ for $BKP3 \geq E \geq BKP4$.
•	•		•
•	•		•
•	•		•
4695	CON(5, 5, 4)		CON(1-25): $C_1, C_2, C_3 \dots C_{25}$ for $E \geq BKP4$.
4720	XLAM	(30)	Values of $\lambda(E_n)$ for each neutron energy group for use in extrapolating either the monovariate or bivariate neutron spectra data for values of $W \geq 120.0 \text{ gm/cm}^2$.

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
	RSPG	(30, 10)	Group-dependent gamma ray response functions. The user must input at least one set of data. If collided and uncollided <u>energy flux</u> is desired as output data, one set of RSPG values must be input as 1.0. The program will provide NRSPG sets of data, and the sum or total over NGG groups for each response function.
4750	RSPG(k, 1)		Response function No. 1, for each energy group, k.
4780	RSPG(k, 2)		Response function No. 2, for each energy group, k.
.	.		.
.	.		.
.	.		.
5020	RSPG(k, 10)		Response function No. 10, for each energy group, k.
	RSPN	(30, 10)	Group dependent response functions for the neutron spectra data. The user must input at least one set of data. If the total (sum over groups) response is desired, (i.e., total neutrons/cm ² -sec) the energy width for each group must appear in RSPN or in the neutron group source, NSOUR.
5050	RSPN(i, 1)		RSPN is input in the same order as RSPG starting with address, 5050.
5350	RSPA	(1, 10)	Response functions for the Albert-Welton function. The user must input at least one value of RSPA (i.e., RSPA (1) = 1.0). The program will provide NRSPA values of output.
	RCORD	(3, 25)	Detector point coordinates (R_D , Z_D , θ_D) for NDET detector points. <u>The detector points must not lie on a boundary of a zone.</u> A maximum of 25 detector points per problem are permitted.
5360	RCORD(i, 1)		(R_D , Z_D , θ_D), for detector No. 1.
5363	RCORD(i, 2)		(R_D , Z_D , θ_D), for detector No. 2.

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
5369	RCORD(i, 3)		(R_D , Z_D , θ_D), for detector No. 3. • • • •
5435	RCORD(i, 25)		(R_D , Z_D , θ_D), for detector No. 25.
5510	SSOT	(3)	Source region translation coordinates (X_T , Y_T , Z_T). The values of SSOT may be used to translate the source region in the problem geometry so that the input source data can be relative to (0, 0, 0).

THE FOLLOWING INPUT DATA ARE REQUIRED ONLY IF MATL IS GREATER THAN ZERO

5513	ZAT	(20)	Atomic number (electrons per atom) of each element for which gamma ray absorption coefficients, are to be calculated by the program. The calculated values, in units of cm^2/gm , will appear as the first MATL sets of gamma ray absorption coefficients. If coefficients are input in conjunction with calculated values, then the input values must be the MATL + 1 to MATL sets of values.
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THE FOLLOWING INPUT DATA ARE REQUIRED ONLY IF LSIT IS GREATER THAN 0

5533	RSIT	(21)	Radial coordinates of source distribution data to be used in the source interpolation routine. There are LSIT values required. The range of RSIT should be greater than or equal to the range of the radial values, RS, so that only interpolation of data is used.
5554	ZSIT	(21)	Axial coordinates of the source distribution data to be used in the source interpolation routine. There are MSIT values required. The range of ZSIT should be greater than or equal to the range of the axial values, ZS, so that only interpolation is used.
	FSIT(i, , k)	(21, 2, 2)	Source distribution data to be used in the source interpolation routine. There are LSIT and MSIT values required. Source interpolation is calculated for both gamma ray and neutron source distributions.

<u>Address</u>	<u>Data</u>	<u>Array Dimension</u>	<u>Description</u>
5575	FSIT(i,1,1)		Radial gamma ray source data.
5596	FSIT(i,2,1)		Axial gamma ray source data.
5617	FSIT(i,1, 2)		Radial neutron source data.
5638	FSIT(i, 2, 2)		Axial neutron source data.

SECTION

5.0 OUTPUT DATA FORMAT

The KAP-V output data is dependent upon the input control words IOUT (1) and IOUT (2) entered at address 32 and 33, respectively. The control word, IOUT (1), controls the print out of the input data. The control word, IOUT (2) controls the print out of the output data.

5.1 INPUT DATA PRINT OUT

The print out of the input data will be described first. If IOUT (1) is set at zero, no input data will be printed, except the program title. This is followed by the output data described in Section 5.2.

If IOUT (1) is set equal to one, the program title is printed, and is followed by a print out of the image of each input data card as used by the computer. Therefore, only columns 1 to 72 are printed. One exception must be noted: If the floating point data, input at address 2060, includes the surface equation constant, D, for equation types 1, 2, or 3, the printed value in the card image is D^2 instead of D. The card image output is followed by the output data described in Section 5.2.

If IOUT (1) is set equal to two, the input data is printed out as described for IOUT (1) equal to one plus a set of labeled input data. This labeled print out is self explanatory (See the sample problem print). Included in the labeled print out are the normalized source distribution data. The labeled print out of the input is followed by the output described in Section 5.2.

5.2 OUTPUT DATA PRINT OUT

The output print out is obviously dependent upon the types of material attenuation functions that are requested in a particular problem. The output is also dependent upon the ISUM Control (address 31), and the IOUT (2) Control (address 33).

If IOUT (2) is set equal to zero, output is printed for each individual source region in the problem. If IOUT (2) is equal to one, the output for each source region is not printed out. (A NOTE OF CAUTION: If the problem contains only one source region, IOUT (2) must be set equal to zero in order to obtain any answers).



The control word, (ISUM), controls the subtotal output over various source regions, and the grand total output over all source regions, as described in the input data instructions (Section 4.0).

If a gamma ray calculation is performed, the collided fluxes multiplied by each set of response functions is printed for the first detector point, for each gamma ray group, as well as the total. This is followed by the uncollided gamma ray data.

If an Albert-Welton calculation is performed, the output multiplied by the response functions follows the gamma ray output data.

If neutron spectra are calculated, the neutron spectra data multiplied by the response functions are then printed.

At the end of the output data for each detector point, a comment is printed which tells the program user how many times the value of 20.0 mean free paths, for gamma rays, or 120.0 gm/cm^2 , for neutrons, was exceeded for a source region.

A sample printout is included with the sample problem.

SECTION

6.0 SAMPLE PROBLEM

6.1 DESCRIPTION

A sample problem is included here to illustrate more clearly the input and output data of the KAP-V program.

The geometry and material description for the sample problem are shown in Figure 8. The problem consists of three material cylindrical regions (Zones I, II, and III), and three outside regions (Zones IV, V, and VI).

The single cylindrical source region is Zone I. The source distribution which is the same for both neutrons and gamma rays is described as uniform in both the radial and axial directions. Variable azimuthal source spacing is employed. Thirteen gamma ray source energy groups describe the source. Gamma ray total absorption coefficients are requested from the built-in library. Water dose buildup factors are input to the program.

The monovariant neutron spectrum option is employed with carbon as the reference material.

Two sets of response functions for both gamma ray data and neutron data are employed.

The problem consists of five detector points as indicated by D₁, D₂, etc. in Figure 8.

6.2 INPUT DATA

The actual FORTRAN input data sheets for the sample problem are given in Table 3. The letters in columns 73 through 80 for the fixed and floating point data refer to the FORTRAN symbol described previously in Section 4.0.

6.3 OUTPUT DATA

The printed output data from a KAP-V problem (CDC 6600 computer job) for the sample problem is presented in Table 4.

The first three pages of output are the card images. Pages 4 through 14 are the labeled input data. This labeled input data is helpful in locating errors or as a permanent



MATERIAL COMPOSITION TABLE

DESCRIPTION	ELEMENT	H	Li	Be	C	Cr	Fe	Ni	U
MATERIAL NO.	COMPOSITION NO.	1	2	3	4	5	6	7	8
CORE	1	0	0	0	0.5	0	0	0	1.69
SIDE REFLECTOR	2	0.007*	0	1.65	0	0	0	0	0
TOP SHIELD	3	0.0514	0.262	0	0	0.256	2.37	0.576	0
ZONE IV	4	0	0	0	0	.0000001	0	0	0
ZONE V	5	0	0	0	0	0	.0000001	0	0
ZONE VI	6	0	0	0	0	0	0	.0000001	0

*UNITS: GM/CM³

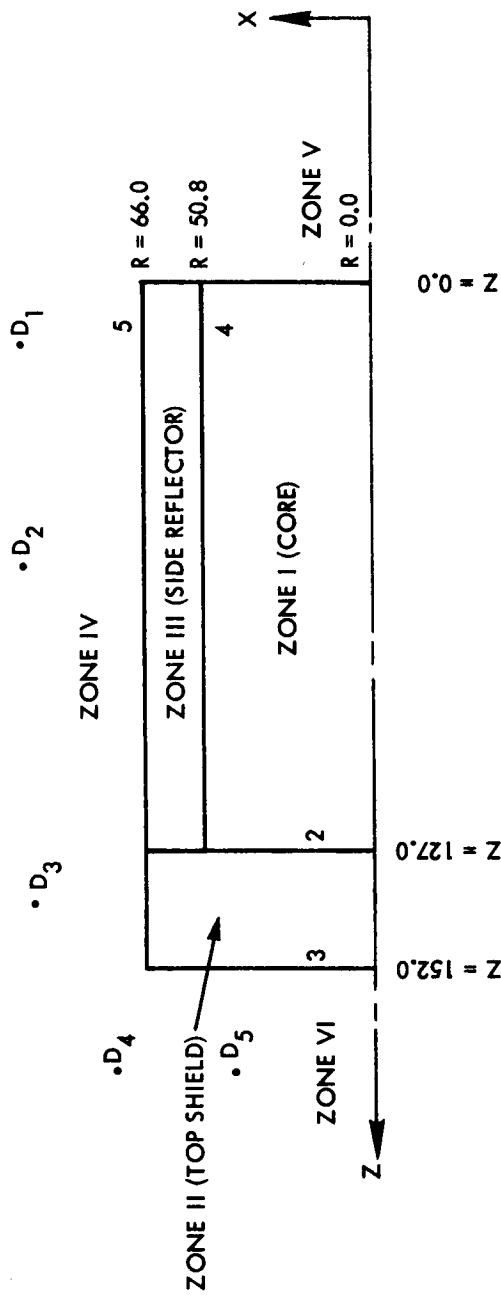


Figure 8. Geometry and Material Compositions for the Sample Problem

611855-468

record of the input used for a particular problem. The normalized source data are included on pages 5 through 8. The gamma ray absorption coefficients (cm^2/gm) obtained from the program library are listed on pages 10 and 11. Notice that, on page 8, the region ambiguity indices which are computed within the program, are printed out.

The calculated output data printout begins on page 14. The column labeled GAM RESP 1, is the gamma ray flux multiplied by the first set of gamma ray response functions which yield units of R/hr. The column labeled GAM RESP 2, is the gamma ray flux multiplied by the second set of gamma ray response functions which yield units of watts/gm of steel.

The column labeled NEUT.1 is the neutron spectrum multiplied by the first neutron response function. The units of NEUT.1 are rads (tissue)/hr. These units were made compatible with the differential neutron spectra data by inputting the NSOUR data as $\Delta E \times 2.46$ neutrons/fission. The column labeled NEUT.1 is the neutron spectrum multiplied by the second response function, and the units, therefore, are $n/\text{cm}^2\text{-sec}$. The remainder of the printout is self-explanatory.



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TABLE 3. SAMPLE PROBLEM INPUT SHEETS

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FORTRAN - CODING FORM

TITLE SAMPLE PROBLEM	ANALYST TITLE DATA	PHONE	DATE	PROGRAM NO.	DECK NO.	L.S. NO.	NORM. NO.	
							1	6
FORTAN STATEMENT								
REF. 1 2 3 4 5 0	7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80							
15	11	NOCKUP FOR GRAPHITE MODERATED CORE, FIVE DETECTOR POINTS,						
15	16	ONE CORE REGION, THREE EXTERNAL REGIONS, SIX INCH SIDE REFL						
15	31	FACTOR, ONE FOOT TOP SHIELD						
15	46	SOURCE REGION CALCULATIONAL RESULTS						
15	76	SUBTOTAL OVER SELECTED SOURCE						
15	106	SUMMATION OVER ALL SOURCE REGIONS						
3	136 GAM RESP 1							
3	139 GAM RESP 2							
3	166 INENT. 1							
3	226 DET. PT. ONE							
3	229 DET. PT. TWO							
3	232 DET. PT. THREE							
3	233 DET. PT. FOUR							
31	238 DET. PT. FIVE							



FORTRAN - CODING FORM

TITLE SAMPLE PROBLEM	ANALYST'S FORTRAN STATEMENT							PHONE	DATE	PROGRAM NO.	DICK NO.	L.S. NO.	SHEET NO. 2 OF 6	
	STAT. NO.	1	2	3	4	5	6							
REF. 1 2 3 4 5	7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80													NECK SEQ. NO.
1 1	1	1	1	1	1	1	1	1	1	1	1	1	1	C 61 TR 2-5
1 4	2 0	1	0	1	0	0	1	1	1	2	0	0	0	OPTIONS
7	5 0	5	1	2	4	6	8	1	0	10	-	-	-	SOURCE
5	1 0 0	6	6	6	3	3	-	-	-	-	-	-	-	NE 9 BD
6	2 0 0	3	3	1	4	-	1	-	1	-	-	-	-	N BND 2 N
6	3 0 0	1	3	1	2	4	5	6	-	-	-	-	-	N C N P Z N
3	5 0 0	1	2	1	4	-	-	-	-	-	-	-	-	LBD 1
3	1 1 0 0	5	2	3	1	-	-	-	-	-	-	-	-	NTRYZN1
3	5 0 6	3	3	1	5	-	-	-	-	-	-	-	-	LBD 2
3	1 1 0 6	1	6	4	-	-	-	-	-	-	-	-	-	NTRYZN2
4	5 1 2	1	4	2	5	-	-	-	-	-	-	-	-	LBD 3
4	1 1 1 2	5	1	3	4	-	-	-	-	-	-	-	-	NTRYZN3
3	5 1 8	1	5	3	-	-	-	-	-	-	-	-	-	LBD 4
3	1 1 1 8	5	3	1	6	-	-	-	-	-	-	-	-	NTRYZN4
1	5 2 4	1	-	-	-	-	-	-	-	-	-	-	-	LBD 5
1	1 1 2 4	1	-	-	-	-	-	-	-	-	-	-	-	NTRYZN5
1	5 3 0	3	-	-	-	-	-	-	-	-	-	-	-	LBD 6
1	1 1 3 0	2	-	-	-	-	-	-	-	-	-	-	-	NTRYZN6



FORTRAN - CODING FORM

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REF.	TITLE	SAMPLE PROBLEM		PHONE	DATE	PROGRAM NO.	DECK NO.	L.S. NO.	SHELL NO.	SEQ. NO.
		STAT. NO.	ANALYST							
1	7 6 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80									
2		1 1.0	0.0							
5		7 0.0	1 0.0							
1		1 2 50.8								
5		281 0.0	15.21		30.5		38.1		50.8	
5		331 6.3.5	76.1		89.0	1	101.6		114.1	
3		381 118.0	120.0		127.0				125	
5		491 0.0	.785		1.57		2.358		3.14	
5		701 0.0	.524		1.048		1.57		2.05	
2		751 2.6.2	3.14						PWI2	
5		911 0.0	.3925		.785		1.179		1.57	
4		961 1.9.6	2.36		2.73		3.14		PWI3	
5		1121 0.0	.314		.628		.942		1.255	
5		1171 1.57	1.88		2.20		2.51		PWI4	
1		1221 3.14							PWI4	
5		1331 0.0	.314		.628		.942		1.255	
5		1381 1.57	1.88		2.2		2.51		PWI5	
1		1431 3.14							PWI5	
5		14001 1.1481	-081 4.347	-081 2.9571	-081 4.3661		-081 2.354		-081 G504R	
5		14051 3.791	-081 2.213	-081 1.8041	-071 4.621		-081 6.997		-081 G504R	
3		14101 4.4591	-071 5.672	-081 1.7291	-071				G504R	
5		14301 9.781	1 9.781	1 7.3351	1 4.891		1 4.891		NSΦUR	
5		14351 3.6681	2.445	2.445	2.445		2.017		1.381	
5		17601 0.0	0.0	0.0	0.0		0.0		XΦBD	
5		18601 0.0	0.0	0.0	0.0		0.0		YΦBD	
5		20601 0.0	127.0	152.0	150.8		66.0		DBD	
3		21601 1.0	0.0	10.0					XYZ	
3		21631 1.0	1 0.0	1 39.81	1				XYZ	



FORTRAN - CODING FORM

REF.	SAMPLE	PROBLEM	ANALYST												DATE	PROGRAM NO.	TRK NO.	L.S. NO.	SHEET NO.	SEQ. NO.	
			FLOATING POINT DATA																		
1 2 3 4 5	7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80																				
3	21 66 1	60 0	0.0																	X Y Z	
3	21 69 1	100 0	0.0																	X Y Z	
3	21 72 1	1.0	0.0																	X Y Z	
3	21 75 1	1.0	0.0																	X Y Z	
5	24 60 1																		CΦMP		
1	24 67 1	1.69																	CΦMP		
5	24 80 1	7.0	-03 0.0																CΦMP		
3	24 85 1																		CΦMP		
5	25 00 1	5.14	-02 262																.256		
3	25 05 1	2.37																	CΦMP		
5	25 20 1																		CΦMP		
1	25 45 1	1.0	-07																1.0	-07 CΦMP	
1	25 66 1	1.0	-07																CΦMP		
5	35 30 1	.6	.1																CΦMP		
3	35 35 1	.021 1	.019																X SEC N		
1	35 50 1	.0407																	X SN REF		
5	35 60 1	8.5																	ENG		
5	35 65 1	3.5	1																ENG		
3	35 70 1	1.0																	ENG		
4	41 90 1	.0051																	ENG		
4	41 94 1	1.004																	BILD		
4	41 98 1	1.003																	BILD		
4	42 02 1	1.002																	BILD		
4	42 06 1	1.001																	BILD		
4	42 10 1	.999 1																	BILD		
4	42 14 1	.997 4																	BILD		
4	42 18 1	.9966 1																	BILD		



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FORTRAN - CODING FORM

TITLE SAMPLE PROBLEM	ANALYST FLOATING POINT DATA										L.S. NO.	DECK NO.	SEQ. NO.	DECK OF
	STAT. NO.	REF.	FORTRAN STATEMENT	PHONE	DATE	PROGRAM NO.	DECK NO.	L.S. NO.	DECK NO.	SEQ. NO.				
1 2 3 4 5	7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80	4	42221 . 9954	. 8357	. 3831	-6. 961	-0.4				BILD			
4	42261 . 9948	. 928	. 07471	-1. 043	-0.3						BILD			
4	42301 . 9968	. 987	. 1714	-9. 766	-0.4						BILD			
4	42341 1. 001	. 9502	. 3278	1. 71	-0.3						BILD			
4	42381 . 9944	1. 069	. 7272	2. 634	-0.2						BILD			
1	43141 60. 0	1	1	1	1	1	1	1	1	1	8KP			
5	43201 2. 2433	-0.5	-2. 6957	-0.3	1. 0613	-0.1	-1. 6947	-1	-4. 5732	C0M				
5	43251 1. 82	-0.5	-2. 1712	-0.3	8. 4253	-0.2	-1. 3253	-1	-3. 4433	C0M				
5	43301 1. 1787	-0.5	-1. 4116	-0.3	5. 577	-0.2	-9. 0566	-0.1	-2. 3533	C0M				
5	43351 9. 022	-0.6	-1. 0811	-0.3	4. 2464	-0.2	-6. 692	-0.1	-1. 8333	C0M				
5	43401 6. 743	-0.6	-8. 0525	-0.4	3. 1311	-0.2	-5. 105	-0.1	-1. 3333	C0M				
5	43451 3. 968	-0.6	-4. 594	-0.4	1. 7025	-0.2	-2. 814	-0.1	-8. 0999	C0M				
5	43501 2. 099	-0.6	-2. 4188	-0.4	8. 7182	-0.3	-1. 5642	-0.1	-6. 5666	C0M				
5	43551 1. 846	-0.6	-1. 9897	-0.4	5. 9974	-0.3	-8. 1066	-0.2	-4. 7666	C0M				
5	43601 2. 64	-0.8	-3. 806	-0.6	-6. 236	-0.4	2. 4483	-0.2	-3. 5333	C0M				
5	43651 -8. 848	-0.7	1. 108	-0.4	-5. 2793	-0.3	9. 8162	-0.2	-3. 7333	C0M				
5	44701 +1. 219	-1.0	5. 0192	-0.8	8. 487	-0.5	-8. 316	-0.2	-10. 894	C0M				
5	44751 -3. 0577	-1.0	4. 1235	-0.7	-1. 59142	-0.4	-1. 947	-0.2	-1. 005	C0M				
5	44801 -3. 9241	-1.0	5. 0637	-0.7	-1. 9489	-0.4	-1. 367	-0.2	-7. 6239	C0M				
5	44851 3. 2355	-1.0	-3. 1185	-0.7	9. 487	-0.5	-4. 6536	-0.2	-4. 848	C0M				
5	44901 1. 0893	-1.0	-1. 1754	-0.7	4. 661	-0.5	-4. 4353	-0.4	-3. 2356	C0M				
5	44951 1. 933	-1.1	-1. 2622	-0.8	1. 497	-0.5	-4. 5817	-0.2	-1. 6727	C0M				
5	45001 3. 182	-1.1	-2. 6598	-0.8	2. 772	-0.5	-5. 1756	-0.2	-6. 649	C0M				
5	45051 -1. 2936	-1.0	1. 45164	-0.7	-2. 848	-0.5	-4. 7853	-0.2	1. 6921	C0M				
5	45101 -5. 541	-1.1	1. 161	-0.7	-3. 916	-0.5	-4. 5128	-0.2	1. 1947	C0M				
5	45151 -1. 05555	-1.0	1. 8044	-0.7	-6. 5216	-0.5	-4. 2916	-0.2	1. 7558	C0M				



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FORTRAN - CODING FORM

TITLE SAMPLE PROBLEM	ANALYSIS FLOATING POINT DATA			PHONE	DATE	PROGRAM NO.	DECK NO.	L.S. NO.	SHUT 6 ON 6
	STAT. NO.	REF.	FORTRAN STATEMENT						
5	47201	0.273	-0.245	.027		.0282		.0305	XLM
5	47251	0.322	-0.332	.0332		.0332		.0332	XLM
5	47501	8.5	-0.7 8.8	-0.7 9.2		-0.7 9.5		-0.7 1.05	-0.7 RSPG
5	47551	1.15	-0.6 1.23	-0.6 1.27		-0.6 1.37		-0.6 1.48	-0.6 RSPG
3	47601	1.61	-0.6 1.67	-0.6 1.66		-0.6 1.66		-0.6 1.66	RSPG
5	47801	3.91	-1.5 3.83	-1.5 3.77		-1.5 3.69		-1.5 3.62	-1.5 RSPG
5	47851	3.59	-1.5 3.62	-1.5 3.65		-1.5 3.70		-1.5 3.88	-1.5 RSPG
3	47901	4.22	-1.5 4.52	-1.5 5.45		-1.5 5.45		-1.5 5.45	RSPG
5	50501	1.97	-0.5 1.94	-0.5 1.78		-0.5 1.71		-0.5 1.62	-0.5 RSPN
5	50551	1.49	-0.5 1.27	-0.5 1.08		-0.5 8.72		-0.6 4.40	-0.6 RSPN
5	50801	1.0	-1.0	-1.0		-1.0		-1.0	RSPN
5	50851	1.0	-1.0	-1.0		-1.0		-1.0	RSPN
3	53601	123.0	-15.2	-0.0		-0.0		-0.0	RCPRD
3	53631	22.0	-91.6	-0.0		-0.0		-0.0	RCPRD
3	53661	122.0	-168.0	-0.0		-0.0		-0.0	RCPRD
3	53691	91.6	-228.0	-0.0		-0.0		-0.0	RCPRD
3	53721	30.5	-228.0	-0.0		-0.0		-0.0	RCPRD
3	55101	0.0	-0.0	-0.0		-0.0		-0.0	SSDT
5	55131	1.0	-3.0	-4.0		-6.0		24.0	ZAT
31	55181	26.0	-28.0	-92.0		-92.0		-92.0	ZAT

TABLE 4. SAMPLE PROBLEM PRINT OUT

MOCKUP FOR GRAPHITE MODERATED CORE, FIVE DETECTOR POINTS, ONE CORE REGION, THREE EXTERNAL REGIONS, SIX INCH SIDE REFL									
11-0	1	13	10	8	6	5	6	2	2
14-0	20	1	0	1	0	0	1	1	0
7-0	50	5	12	4	6	8	10	10	0
2-0	75	0	0	0	0	0	0	0	0
5-0	100	6	6	6	3	3	4	4	0
6-0	200	3	3	4	-1	-1	-1	-1	0
6-0	300	1	3	2	4	5	6	6	0
3-0	500	1	2	3	4	5	6	6	0
3-0	1000	5	2	3	4	5	6	6	0
3-0	1500	2	3	5	4	5	6	6	0
3-0	1506	2	3	5	4	5	6	6	0
3-0	1106	1	6	4	5	6	6	6	0
3-0	1106	1	6	4	5	6	6	6	0
4-0	512	1	4	2	5	6	6	6	0
4-0	1112	5	1	2	4	5	6	6	0
3-0	518	1	5	3	4	5	6	6	0
3-0	1118	5	3	6	4	5	6	6	0
1-0	524	1	1	1	0	0	0	0	0
1-0	1124	1	1	1	0	0	0	0	0
1-0	530	3	1	1	0	0	0	0	0
1-1	1130	2	1	1	0	0	0	0	0
2-0	1	1	1	0	0	0	0	0	0
5-0	7	0	0	0	0	0	0	0	0
1-0	12	5.0000E+01	1.0000E+01	2.0000E+01	3.0000E+01	4.0000E+01	5.0000E+01	6.0000E+01	7.0000E+01
3-0	38	1.1800E+02	1.2000E+02	1.2700E+02	3.0500E+01	3.8100E+01	4.0160E+02	4.1410E+02	4.1410E+02
5-0	28	0	1.5210E+01	0	0	0	0	0	0
5-0	33	6.3500E+01	7.6100E+01	8.9000E+01	1.5700E+00	2.3580E+00	3.1400E+00	3.1400E+00	3.1400E+00
5-0	49	0	0	0	0	0	0	0	0
5-0	70	0	5.2400E+01	1.0480E+00	1.0480E+00	1.5700E+00	2.0950E+00	2.0950E+00	2.0950E+00
2-0	75	2.62000E+00	3.1400E+00	3.9250E+01	7.8500E+01	1.1790E+00	1.5700E+00	1.5700E+00	1.5700E+00
5-0	91	0	0	2.3600E+00	2.7300E+00	3.1400E+00	3.1400E+00	3.1400E+00	3.1400E+00
4-0	96	1.9600E+00	3.1400E+00	6.2800E+01	9.4200E+01	9.4200E+01	1.2550E+00	1.2550E+00	1.2550E+00
5-0	112	0	0	1.8800E+00	2.2000E+00	2.5100E+00	2.8200E+00	2.8200E+00	2.8200E+00
5-0	117	1.57000E+00	1.8800E+00	2.2000E+00	2.5100E+00	2.8200E+00	2.8200E+00	2.8200E+00	2.8200E+00
1-0	122	3.14000E+00	3.14000E+00	3.14000E+01	6.28000E+01	9.42000E+01	1.25500E+00	1.25500E+00	1.25500E+00
5-0	133	0	0	1.8800E+00	2.2000E+00	2.5100E+00	2.8200E+00	2.8200E+00	2.8200E+00
1-0	136	1.57000E+00	1.8800E+00	2.2000E+00	2.5100E+00	2.8200E+00	2.8200E+00	2.8200E+00	2.8200E+00
1-0	143	3.14000E+00	4.34700E+08	2.95700E+08	2.95700E+08	4.36600E+08	4.36600E+08	4.36600E+08	4.36600E+08
1-0	1400	1.14800E+08	2.21300E+08	1.80400E+08	1.80400E+08	4.16200E+08	4.16200E+08	4.16200E+08	4.16200E+08
5-0	1405	3.79100E+08	5.67700E+07	1.72900E+07	1.72900E+07	6.99700E+08	6.99700E+08	6.99700E+08	6.99700E+08
3-0	1410	4.45900E+07	0	0	0	0	0	0	0
5-0	1430	9.78000E+00	9.78000E+00	7.33500E+00	4.89000E+00	4.89000E+00	4.89000E+00	4.89000E+00	4.89000E+00
5-0	1435	3.66800E+00	2.44500E+00						
5-0	1760	0	0	0	0	0	0	0	0
5-0	1860	0	0	0	0	0	0	0	0

5-0	2060	0.	1.27000E+02	1.52000E+02	2.58064E+03	4.35600E+03
3-0	2160	1.00000E+00	0.	1.00000E+01	1.39800E+01	
3-0	2163	1.00000E+00	0.	1.00000E+02		
3-0	2166	6.00000E+01	0.	1.00000E+02		
3-0	2169	1.00000E+02	0.	1.00000E+01		
3-0	2172	1.00000E+00	0.	-1.00000E+01		
3-0	2175	1.00000E+00	0.	1.70000E+02		
5-0	2460	-0.	-0.	-0.	5.00000E-01	-0.
1-0	2467	1.69000E+00	-0.	1.65000E+00	-0.	-0.
1-0	2480	7.00000E-03	0.	7.0.		
3-0	2485	-0.	-0.	-0.	-0.	2.56000E+01
5-0	2500	5.14000E-02	2.62000E-01	-0.		
3-0	2505	2.37000E+00	5.76000E-01	-0.		
5-0	2520	-0.	-0.	-0.	-0.	1.00000E+07
1-0	2545	1.00000E-07				
1-0	2566	1.00000E-07				
5-0	3530	6.00000E-01	1.00000E-01	7.20000E-02	4.07000E-02	2.17000E-02
3-0	3535	2.10000E-02	1.90000E-02	9.10000E-03		
1-0	3550	4.07000E+02	7.25000E+00	6.50000E+00	5.50000E+00	4.50000E+00
5-0	3560	8.50000E+00	2.80000E+00	2.50000E+00	2.00000E+00	1.50000E+00
5-0	3565	3.50000E+00	7.00000E+00	3.00000E+01		
3-0	3570	1.00000E+00	3.61600E+01	6.49000E+03	1.35400E+04	
4-0	4190	1.00000E+00	3.90200E+01	-5.65200E+03	1.16200E+04	
4-0	4194	1.00400E+00	4.26300E+01	-4.65000E+03	9.19700E+05	
4-0	4198	1.00300E+00	4.73000E+01	-3.18500E+03	4.97500E+05	
4-0	4202	1.00200E+00	5.35700E+01	-1.79500E+04	-2.26500E+05	
4-0	4206	1.00100E+00	6.23600E+01	5.93800E+03	-1.58100E+04	
4-0	4210	9.99000E+01	7.07800E+01	1.47900E+02	-3.31700E+04	
4-0	4214	9.97400E+01	7.07800E+01	1.47900E+02	-3.31700E+04	
4-0	4218	9.96600E+01	7.51600E+01	2.19400E+02	-4.41400E+04	
4-0	4222	9.95400E+01	8.35700E+01	3.86300E+01	-6.96100E+04	
4-0	4226	9.94800E+01	9.28000E+01	7.47100E+02	-1.04300E+03	
4-0	4230	9.96800E+01	9.87000E+01	1.71400E+01	-9.76600E+04	
4-0	4234	1.00100E+00	9.50200E+01	3.27800E+01	1.71000E+03	
4-0	4238	9.94400E+01	1.06900E+00	7.27200E+01	2.63400E+02	
4-0	4314	6.00000E+01				
5-0	4320	2.24330E+05	-2.69570E+03	1.06130E+01	-1.69470E+00	-4.57320E+00
5-0	4325	1.92000E+05	-2.17120E+03	8.42530E+02	-1.32530E+00	-3.44330E+00
5-0	4330	1.17870E+05	-1.41600E+03	5.57700E+02	-9.05660E+01	-2.35330E+00
5-0	4335	9.02200E+04	-1.08110E+03	4.24640E+02	-6.96920E+01	-1.63330E+00
5-0	4340	6.74300E+04	-8.05250E+04	3.13110E+02	-5.10500E+01	-1.33330E+00
5-0	4345	3.96800E+04	-4.59940E+04	1.70240E+02	-2.81400E+01	-8.09990E+01
5-0	4350	2.09900E+04	-2.41680E+04	8.71820E+03	-1.56420E+01	-6.56660E+01
5-0	4355	1.84600E+04	-1.98970E+04	5.9740E+03	-8.1060E+02	-4.76660E+01
5-0	4360	2.64000E+04	-3.80600E+04	6.23600E+04	-2.4830E+02	-3.53330E+01
5-0	4365	-8.84800E+07	1.10800E+04	-5.27930E+03	9.81620E+02	-3.73330E+01



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5-0	4470	-1.2190E-10	5.01920E-08	8.48700E-05	-8.31600E-02	-1.08940E+01
5-0	4475	-3.0577E-10	4.12350E-07	-1.59420E-04	-1.94700E-02	-1.10050E+01
5-0	4480	-3.92410E-10	5.06370E-07	-1.94890E-04	-1.36700E-02	-7.62390E+00
5-0	4485	3.23550E-10	-3.11850E-07	9.48700E-05	-4.65360E-02	-4.84800E+00
5-0	4490	1.08930F-10	-1.17540E-07	4.66100E-05	-4.43530F-02	-3.23560E+00
5-0	4495	1.93300F-11	-1.26220E-08	1.49700E-05	-4.58700F-02	-1.67270E+00
5-0	4500	3.18200F-11	-2.65900E-08	2.77200E-05	-5.17560F-02	-6.64900E-01
5-0	4505	-1.29360E-10	1.45640E-07	-2.84800E-05	-4.78930F-02	1.69210E-01
5-0	4510	-5.54100E-11	1.16100F-07	-3.99600E-05	-4.51280E-02	1.19470E+00
5-0	4515	-1.05550E-10	1.80440E-07	-6.52600E-05	-4.29600F-02	1.75580E+00
5-0	4720	2.73000E-02	2.45000F-02	2.70000E-02	2.82000F-02	3.05000E-02
5-0	4725	3.22000E-02	3.32000E-02	3.32000E-02	3.32000E-02	3.32000E-02
5-0	4750	8.50000E-07	8.80000E-07	9.20000E-07	9.50000E-07	1.05000E-06
5-0	4755	1.15700F-06	1.23000E-06	1.27000E-06	1.37000F-06	1.48000E-06
3-0	4760	1.61000E-06	1.67000E-06	1.66000E-06		
5-0	4780	3.91000F-15	3.83000E-15	3.77000E-15	3.69000F-15	3.62000E-15
5-0	4785	3.59000E-15	3.62000E-15	3.65000E-15	3.70000F-15	3.88000E-15
3-0	4790	4.22000F-15	4.52000E-15	5.45000E-15		
5-0	5050	1.97000E-05	1.94000E-05	1.78000E-05	1.71000F-05	1.62000E-05
5-0	5055	1.49000E-05	1.27000E-05	1.08000E-05	8.72000F-06	4.40000E-06
5-0	5060	1.00000E+00	1.00000E+00	1.00000E+00	1.00000E+00	1.00000E+00
5-0	5085	1.00000E+00	1.00000E+00	1.00000E+00	1.00000E+00	1.00000E+00
3-0	5360	1.22000E+02	1.52000E+01	1.		
3-0	5363	1.22000E+02	9.16000E+01	0.		
3-0	5366	1.22000E+02	1.68000E+02	0.		
3-0	5369	9.16000E+01	2.28000E+02	0.		
3-0	5372	3.05000F+01	2.28000E+02	0.		
3-0	5510	0.	0.	0.		
5-0	5513	1.00000E+00	3.00000E+00	4.00000E+00	6.00000F+00	2.40000E+01
5-0	5518	2.60000E+01	2.80000E+01	9.20000E+01		

GENERAL PROBLEM INPUT

```

NO. OF GAMMA GROUPS..... 13
NO. OF NEUTRON GROUPS... 1
NO. OF MATERIALS..... 2
NO. OF COMPOSITIONS.... 2
NO. OF DETECTORS.... 4
NO. OF BOUNDARIES.... 5
NO. OF REGIONS..... 6
NO. OF RESPONSES(GAMMA) .. 2
NO. OF RESPONSES(NEUTRON) .. 2
NO. OF RESPONSES(ALBERT-WELTON) .. 0
NO. OF MATERIALS FROM LIBRARY.. 0

```

CALCULATION OPTIONS

```

GAMMA RAY(0/NO.1/YES) .. 1
NEUTRON(0/NO.1/YES) ... 0
ALBERT-WELTON..... 0
MONOVARIANT MOMENTS... 1
BIVARIANT MOMENTS.... 0

```

PRINT OPTIONS

```

SOURCE REGION SUMMATION
(0/NO.1/SUM*2/ST*3/GT)*..... 2
INPUT PRINT(0/NO.1/C1.2/FULL) :
(1/ST AND GT ONLY*2/ALL) ..... 0

```

SOURCE CALCULATION OPTIONS

```

SOURCE ZONE COMPOSITION.... 0
MOST PROBABLE ZONE..... 1
SOURCE CALCULATION OPTION... 1
RADIAL DISTRIBUTION.... 1
AXIAL OR POLAR DISTRIBUTION.. 1
AZMUTHIAL DISTRIBUTION.... 2

```

SOURCE INTERVAL DATA

```

NO. OF RADIAL..... 5

```

NO. OF AXIAL (OR POLAR)..... 12
 NO. OF AZMUTHIAL..... 4 6 8 10 10

SOURCE AND DISTRIBUTION PARAMETERS

GAMMA RAY SOURCE (INPUT)	1.00000E+00
(NORMALIZED)	1.94343E-06
NEUTRON SOURCE (INPUT)	0.
(NORMALIZED)	1.94343E-06
DISTRIBUTION PARAMETERS (X1(1))	0.
(X1(2))	0.
(ETA(1))	0.
(ETA(2))	0.

SOURCE DISTRIBUTION DATA - RADIAL

PT. NO.	COORDINATE INPUT	MIDPOINT	GAMMA RAY INPUT	NORMALIZED	NEUTRON INPUT	NORMALIZED
1	0.	7.071E+00	0.	5.000E+01	0.	5.000E+01
2	1.000E+01	1.581E+01	0.	1.500E+02	0.	1.500E+02
3	2.000E+01	2.550E+01	0.	2.500E+02	0.	2.500E+02
4	3.000E+01	3.536E+01	0.	3.500E+02	0.	3.500E+02
5	4.000E+01	4.572E+01	0.	4.903E+02	0.	4.903E+02
6	5.000E+01	0.				

SOURCE DISTRIBUTION DATA - AXIAL OR POLAR

PT. NO.	COORDINATE INPUT	MIDPOINT	GAMMA RAY INPUT	NORMALIZED	NEUTRON INPUT	NORMALIZED
1	1.521E+01	7.605E+00	0.	1.521E+01	0.	1.521E+01
2	3.050E+01	2.285E+01	0.	1.529E+01	0.	1.529E+01
3	3.610E+01	3.430E+01	0.	7.600E+00	0.	7.600E+00
4	4.000E+01	4.445E+01	0.	1.270E+01	0.	1.270E+01
5	5.000E+01	5.715E+01	0.	1.270E+01	0.	1.270E+01
6	6.350E+01	6.980E+01	0.	1.260E+01	0.	1.260E+01
7	7.610E+01	8.255E+01	0.	1.290E+01	0.	1.290E+01
8	8.900E+01	9.530E+01	0.	1.260E+01	0.	1.260E+01
9	1.016E+02	1.078E+02	0.	1.250E+01	0.	1.250E+01
10	1.141E+02	1.160E+02	0.	3.900E+00	0.	3.900E+00

11	1.1A0E+02	1.190E+02	0.	2.000E+00	0.	2.000E+00
12	1.200E+02	1.235E+02	0.	7.000E+00	0.	7.000E+00
13	1.270E+02		0.			

SOURCE DISTRIBUTION DATA - AZMUTHIAL FOR RADIAL INTERVAL 1

PT. NO.	COORDINATE INPUT	MIDPOINT	GAMMA RAY INPUT	NORMALIZED	NEUTRON INPUT	NORMALIZED
1	0.	3.925E+01	0.	1.526E-06	0.	1.526E-06
2	7.650E-01	1.177E+00	0.	1.526E-06	0.	1.526E-06
3	1.570E+00	1.964E+00	0.	1.531E-06	0.	1.531E-06
4	2.358E+00	2.749E+00	0.	1.520E-06	0.	1.520E-06
5	3.140E+00		0.			

SOURCE DISTRIBUTION DATA - AZMUTHIAL FOR RADIAL INTERVAL 2

PT. NO.	COORDINATE INPUT	MIDPOINT	GAMMA RAY INPUT	NORMALIZED	NEUTRON INPUT	NORMALIZED
1	0.	2.620E+01	0.	1.018E-06	0.	1.018E-06
2	5.240E-01	7.860E-01	0.	1.018E-06	0.	1.018E-06
3	1.048E+00	1.309E+00	0.	1.014E-06	0.	1.014E-06
4	1.570E+00	1.832E+00	0.	1.020E-06	0.	1.020E-06
5	2.095E+00	2.358E+00	0.	1.020E-06	0.	1.020E-06
6	2.620E+00	2.880E+00	0.	1.011E-06	0.	1.011E-06
7	3.140E+00		0.			

SOURCE DISTRIBUTION DATA - AZMUTHIAL FOR RADIAL INTERVAL 3

PT. NO.	COORDINATE INPUT	MIDPOINT	GAMMA RAY INPUT	NORMALIZED	NEUTRON INPUT	NORMALIZED
1	0.	1.963E+01	0.	7.628E-07	0.	7.628E-07
2	3.925E+01	5.887E+01	0.	7.628E-07	0.	7.628E-07
3	7.850E+01	9.820E+01	0.	7.657E-07	0.	7.657E-07
4	1.179E+02	1.374E+02	0.	7.599E-07	0.	7.599E-07
5	1.570E+02	1.765E+02	0.	7.579E-07	0.	7.579E-07
6	1.960E+02	2.160E+02	0.	7.774E-07	0.	7.774E-07
7	2.360E+02	2.545E+02	0.	7.191E-07	0.	7.191E-07

8	2.730E+00	2.935E+00	0.	7.968E+07	0.	7.968E+07
9	3.140E+00	0.				

SOURCE DISTRIBUTION DATA - AZIMUTHIAL FOR RADIAL INTERVAL 4

PT. NO.	COORDINATE INPUT	MIDPOINT	GAMMA RAY INPUT	NORMALIZED	NEUTRON INPUT	NORMALIZED
1	0.	1.570E-01	0.	6.102E-07	0.	6.102E-07
2	3.140E-01	4.710E-01	0.	6.102E-07	0.	6.102E-07
3	6.280E-01	7.850E-01	0.	6.102E-07	0.	6.102E-07
4	9.420E-01	1.099E+00	0.	6.083E-07	0.	6.083E-07
5	1.255E+00	1.413E+00	0.	6.122E-07	0.	6.122E-07
6	1.570E+00	1.725E+00	0.	6.025E-07	0.	6.025E-07
7	1.880E+00	2.040E+00	0.	6.219E-07	0.	6.219E-07
8	2.200E+00	2.355E+00	0.	6.025E-07	0.	6.025E-07
9	2.510E+00	2.665E+00	0.	6.025E-07	0.	6.025E-07
10	2.820E+00	2.980E+00	0.	6.219E-07	0.	6.219E-07
11	3.140E+00	0.	0.	0.	0.	0.

SOURCE DISTRIBUTION DATA - AZIMUTHIAL FOR RADIAL INTERVAL 5

PT. NO.	COORDINATE INPUT	MIDPOINT	GAMMA RAY INPUT	NORMALIZED	NEUTRON INPUT	NORMALIZED
1	0.	1.570E-01	0.	6.102E-07	0.	6.102E-07
2	3.140E-01	4.710E-01	0.	6.102E-07	0.	6.102E-07
3	6.280E-01	7.850E-01	0.	6.102E-07	0.	6.102E-07
4	9.420E-01	1.099E+00	0.	6.083E-07	0.	6.083E-07
5	1.255E+00	1.413E+00	0.	6.122E-07	0.	6.122E-07
6	1.570E+00	1.725E+00	0.	6.025E-07	0.	6.025E-07
7	1.880E+00	2.040E+00	0.	6.219E-07	0.	6.219E-07
8	2.200E+00	2.355E+00	0.	6.025E-07	0.	6.025E-07
9	2.510E+00	2.665E+00	0.	6.025E-07	0.	6.025E-07
10	2.820E+00	2.980E+00	0.	6.219E-07	0.	6.219E-07
11	3.140E+00	0.	0.	0.	0.	0.

GAMMA RAY SPECTRAL DATA

GROUP NO.	ENERGY	SPECTRAL DATA

GROUP NO.	ENERGY	SPECTRAL DATA
1	0.0	9.780E+00
2	0.0	9.780E+00
3	0.0	7.335E+00
4	0.0	4.890E+00
5	0.0	4.890E+00
6	0.0	3.668E+00
7	0.0	2.445E+00
8	0.0	2.445E+00
9	0.0	2.017E+00
10	0.0	1.381E+00

NEUTRON SPECTRAL DATA

ZONE	COMP.	BND.	ZONE								
1	1	-1	5	2	2	4	3				
2	3	-2	1	3	6	5	4				
3	2	-1	5	-4	1	2	5	4			
4	4	-1	5	1	1						
5	5	1	1								
6	6	-3									

ZONE BOUNDARY SPECIFICATIONS

ZONE	COMP.	BND.	ZONE								
1	1	-1	5	2	2	4	3				
2	3	-2	1	3	6	5	4				
3	2	-1	5	-4	1	2	5	4			
4	4	-1	5	1	1						
5	5	1	1								
6	6	-3									

BOUNDARY EQUATION CONSTANTS



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BND. NO.	AND TYPE	A0	X0	B0	Y0	C0	Z0	DO
1	6	0.	0.	0.	0.	0.	0.	0.
2	6	0.	0.	0.	0.	0.	0.	1.270E+02
3	6	0.	0.	0.	0.	0.	0.	1.520E+02
4	3	0.	0.	0.	0.	0.	0.	2.581E+03
5	3	0.	0.	0.	0.	0.	0.	4.356E+03

COMPOSITIONS BY MATERIAL

COMP. NO.	1	2	3	4	5	6
1	-0.	7.000E-03	0.	-0.	5.000E-01	0.
2	5.140E-02	2.620E-01	1.650E+00	-0.	-0.	-0.
3	-0.	-0.	-0.	-0.	2.560E-01	2.370E+00
4	0.	0.	0.	-0.	1.000E-07	0.
5	0.	0.	0.	0.	0.	1.000E-07
6	0.	0.	0.	0.	0.	0.

COMPOSITIONS BY MATERIAL

COMP. NO.	7	8	MATERIAL NO.
1	0.	1.690E+00	
2	-0.	-0.	
3	5.760E-01	-0.	
4	0.	0.	
5	0.	0.	
6	1.000E-07	0.	

NEUTRON CROSS SECTION DATA

MAT. NO.	ALERT-WELTON REMOVAL	ETA	MOMENTS REMOVAL
1	0.	0.	6.000E-01
2	0.	0.	1.000E-01
3	0.	0.	7.200E-02
4	0.	0.	4.070E-02
5	0.	0.	2.170E-02
6	0.	0.	2.100E-02



7 0. 0. 1.900E-02
8 0. 0. 9.100E-03

ALBERT-WELTON COEFF..•ALPHA(1)••••
•ALPHA(2)•••• 0.
•ALPHA(3)•••• 0.
•ALPHA(4)•••• 0.
•ALPHA(5)•••• 0.
•ALPHA(6)•••• 0.
•ALPHA(7)•••• 0.

REFERENCE MATERIAL REMOVAL..••••• 4.070E-02

GAMMA RAY LINEAR ABSORPTION COEFF.

MATERIAL NO.

COMP. NO.	ENERGY	1	2	3	4	5	6
1	0.500E+00	3.550E-02	1.680E-02	1.730E-02	2.050E-02	2.924E-02	2.980E-02
2	7.250E+00	3.969E-02	1.827E-02	1.999E-02	2.227E-02	3.967E-02	3.012E-02
3	6.250E+00	4.260E-02	1.935E-02	2.059E-02	2.360E-02	3.007E-02	3.043E-02
4	5.500E+00	4.720E-02	2.140E-02	2.220E-02	2.560E-02	3.086E-02	3.110E-02
5	4.500E+00	5.350E-02	2.400E-02	2.470E-02	2.840E-02	3.220E-02	3.228E-02
6	3.500E+00	6.290E-02	2.790E-02	2.860E-02	3.270E-02	3.466E-02	3.455E-02
7	2.800E+00	7.250E-02	3.180E-02	3.270E-02	3.720E-02	3.769E-02	3.742E-02
8	2.500E+00	7.750E-02	3.390E-02	3.490E-02	3.950E-02	3.952E-02	3.918E-02
9	2.000E+00	8.760E-02	3.830E-02	3.940E-02	4.440E-02	4.338E-02	4.291E-02
10	1.500E+00	1.030E-01	4.480E-02	4.590E-02	5.180E-02	4.963E-02	4.905E-02
11	1.000E+00	1.260E-01	5.500E-02	5.650E-02	6.360E-02	6.062E-02	5.988E-02
12	7.000E-01	1.480E-01	6.550E-02	6.610E-02	7.450E-02	7.247E-02	7.159E-02
13	3.000E-01	2.120E-01	9.220E-02	9.450E-02	1.060E-01	1.063E-01	1.063E-01

GAMMA RAY LINEAR ABSORPTION COEFF.

MATERIAL NO.

COMP. NO.	ENERGY	7	8
1	0.500E+00	3.038E-02	4.088E-02
2	7.250E+00	3.059E-02	4.0730E-02
3	6.500E+00	3.082E-02	4.0628E-02
4	5.500E+00	3.136E-02	4.0492E-02
5	4.500E+00	3.239E-02	4.0383E-02



GROUP	ENERGY	80	81	82	83
NO.					
1	8.900E+00	1.000E+00	3.616E+01	7.604E+01	1.354E+04
2	7.205E+00	1.004E+00	3.902E+01	-5.052E+03	1.102E+04
3	6.500E+00	1.003E+00	4.293E+01	-4.055E+03	9.167E+05
4	5.500E+00	1.002E+00	4.730E+01	-3.165E+03	4.975E+05
5	4.500E+00	1.001E+00	5.357E+01	-1.795E+04	-2.265E+05
6	3.500E+00	1.000E+00	6.236E+01	5.938E+03	-1.581E+04
7	2.800E+00	9.974E+01	7.078E+01	1.478E+02	-3.317E+04
8	2.500E+00	9.966E+01	7.516E+01	2.104E+02	-4.414E+04
9	2.000E+00	9.954E+01	8.357E+01	3.831E+01	-6.961E+04
10	1.500E+00	9.948E+01	9.280E+01	7.471E+02	-1.043E+03
11	1.000E+00	9.988E+01	9.710E+01	1.714E+01	-9.766E+04
12	7.000E+01	1.001E+00	9.502E+01	3.278E+01	1.710E+03
13	3.000E+01	9.944E+01	1.069E+00	7.212E+01	2.634E+02

GAMMA RAY BUILDUP COEFF.

GROUP	ENERGY	80	81	82	83
NO.					
1	8.900E+00	1.000E+00	3.616E+01	7.604E+01	1.354E+04
2	7.205E+00	1.004E+00	3.902E+01	-5.052E+03	1.102E+04
3	6.500E+00	1.003E+00	4.293E+01	-4.055E+03	9.167E+05
4	5.500E+00	1.002E+00	4.730E+01	-3.165E+03	4.975E+05
5	4.500E+00	1.001E+00	5.357E+01	-1.795E+04	-2.265E+05
6	3.500E+00	1.000E+00	6.236E+01	5.938E+03	-1.581E+04
7	2.800E+00	9.974E+01	7.078E+01	1.478E+02	-3.317E+04
8	2.500E+00	9.966E+01	7.516E+01	2.104E+02	-4.414E+04
9	2.000E+00	9.954E+01	8.357E+01	3.831E+01	-6.961E+04
10	1.500E+00	9.948E+01	9.280E+01	7.471E+02	-1.043E+03
11	1.000E+00	9.988E+01	9.710E+01	1.714E+01	-9.766E+04
12	7.000E+01	1.001E+00	9.502E+01	3.278E+01	1.710E+03
13	3.000E+01	9.944E+01	1.069E+00	7.212E+01	2.634E+02

MONOVARIANT POLYNOMIAL MOMENTS DATA

GROUP	ENERGY	A0	A1	A2	A3	A4
NO.						
1	1.061E+01	-1.695E+00	-4.573E+00			
2	1.061E+01	-1.695E+00	-4.573E+00			
3	2.243E+15	-2.6996E+03	1.061E+01	1.695E+00	-4.573E+00	
4	1.020E+05	-2.01712E+03	8.425E+02	1.3295E+00	-3.443E+00	
5	1.020E+05	-2.01712E+03	8.425E+02	1.3295E+00	-3.443E+00	
6	1.020E+05	-2.01712E+03	8.425E+02	1.3295E+00	-3.443E+00	
7	1.020E+05	-2.01712E+03	8.425E+02	1.3295E+00	-3.443E+00	
8	1.020E+05	-2.01712E+03	8.425E+02	1.3295E+00	-3.443E+00	
9	1.020E+05	-2.01712E+03	8.425E+02	1.3295E+00	-3.443E+00	
10	1.020E+05	-2.01712E+03	8.425E+02	1.3295E+00	-3.443E+00	
11	1.020E+05	-2.01712E+03	8.425E+02	1.3295E+00	-3.443E+00	
12	1.020E+05	-2.01712E+03	8.425E+02	1.3295E+00	-3.443E+00	
13	1.020E+05	-2.01712E+03	8.425E+02	1.3295E+00	-3.443E+00	

5	0.	1.089E-10	-1.175E-07	4.661E-05	-4.435E-02	-3.236E+00
6	0.	1.933E-11	-1.262E-08	1.497E-05	-4.587E-02	-1.673E+00
7	0.	3.182E-11	-2.660E-08	2.772E-05	-5.176E-02	-6.694E-01
8	0.	-1.294E-10	1.456E-07	-2.848E-05	-4.789E-02	1.692E+01
9	0.	-5.541E-11	1.161E-07	-3.996E-05	-4.513E-02	1.195E+00
10	0.	-1.055E-10	1.804E-07	-6.526E-05	-4.296E-02	1.756E+00

BIVARIANT POLYNOMIAL MOMENTS DATA

GROUP NO.	ENERGY RANGE LOWER	UPPER	LIMITS	C0	C1	C2	C3	C4
1	6.000E+01	0.	0.	0.	0.	0.	0.	0.
2	0.	0.	0.	0.	0.	0.	0.	0.
3	0.	0.	0.	0.	0.	0.	0.	0.

MOMENTS DATA EXTRAPOLATION PARAMETERS

GROUP NO.	ENERGY	LAMBDA
1	0.	2.730E-02
2	0.	2.450E-02
3	0.	2.700E-02
4	0.	2.820E-02
5	0.	3.050E-02
6	0.	3.220E-02
7	0.	3.320E-02
8	0.	3.320E-02
9	0.	3.320E-02
10	0.	3.320E-02

GAMMA RAY RESPONSE DATA

GROUP NO.	ENERGY	RESPONSE FUNCTION
1	0.500E+00	8.500E-07
2	7.250E+00	6.800E-07
3	6.500E+00	9.200E-07
4	5.500E+00	9.500E-07
5	4.500E+00	1.050E-06
6	3.500E+00	1.150E-06
7	2.800E+00	1.230E-06
8	2.500E+00	1.270E-06
9	2.000E+00	1.370E-06
10	1.500E+00	1.480E-06

11	1.000E+00	1.610E-06	4.220E-15
12	7.000E-01	1.670E-06	4.520E-15
13	3.000E-01	1.660E-06	5.450E-15

NEUTRON RESPONSE DATA

GROUP NO.	ENERGY	RESPONSE FUNCTION
1	0.	1.000E+00
2	0.	1.970E-05
3	0.	1.940E-05
4	0.	1.780E-05
5	0.	1.710E-05
6	0.	1.620E-05
7	0.	1.490E-05
8	0.	1.270E-05
9	0.	1.080E-05
10	0.	4.400E-06

ALBERT/WELTON RESPONSE DATA

GROUP NO.	ENERGY	RESPONSE FUNCTION
1	1.000E+00	0.

RECEIVER POINT COORDINATES

POINT NO.	X	Y	Z	COORDINATES
1	1.220E+02	1.520E+01	0.	
2	1.220E+02	9.160E+01	0.	
3	1.220E+02	1.680E+02	0.	
4	9.160E+01	2.280E+02	0.	
5	3.050E+01	2.240E+02	0.	

SOURCE POINT TRANSLATION COORDINATES

X.....	0.
Y.....	0.
Z.....	0.

LIBRARY MATERIAL ATOMIC NUMBERS

MAT.	ATOMIC
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NO.	NUMBER
1	1.000E+00
2	3.000E+00
3	4.000E+00
4	6.000E+00
5	2.400E+01
6	2.600E+01
7	2.800E+01
8	9.200E+01

MISCELLANEOUS DATA

BOUNDARY SEARCH EPSILON.....	1.000E-06
BOUNDARY SEARCH PARAMETER....	1.000E-03
EMPIRICAL SOURCE SOLUTION	
MEAN FREE PATHS.....	0.

CALCULATED RESULTS FOR SOURCE REGION 1

SOURCE REGION CALCULATIONAL RESULTS

RECEIVER POINT DET.PT. ONE	
COORDINATES R.....	1.22000E+02
Z.....	1.52000E+01
PHI.....	0.

GAM RESP 1 GAM RESP 2

1	8.50	2.297E+20
2	7.25	9.569E+20
3	6.50	7.442E+20
4	5.50	1.248E+19
5	4.50	8.248E+20
6	3.50	1.594E+19
7	2.80	1.044E+19
8	2.50	8.978E+19
9	2.00	1.435E+18
10	1.50	4.415E+19



Astronuclear
Laboratory

11	1.00	3.060E-18	8.043E-27
12	.70	3.770E-19	1.020E-27
13	.30	4.168E-20	1.368E-28
	
		6.926E-18	1.922E-26

SOURCE REGION CALCULATIONAL RESULTS

RECEIVER POINT DET.PT. ONE
COORDINATES R..... 1.22000E+02
Z..... 1.52000E+01
PHI... 0.

GAM RESP 1 GAM RESP 2

1	8.50	3.416E-21	1.571E-29
2	7.25	1.291E-20	5.618E-29
3	6.50	8.940E-21	3.663E-29
4	5.50	1.302E-20	5.057E-29
5	4.50	7.196E-21	2.481E-29
6	3.50	1.093E-20	3.412E-29
7	2.80	5.668E-21	1.668E-29
8	2.50	4.293E-20	1.234E-28
9	2.00	8.690E-21	2.344E-29
10	1.50	1.074E-20	2.817E-29
11	1.00	3.766E-20	9.870E-29
12	.70	2.288E-21	6.192E-30
13	.30	7.955E-23	2.612E-31
	
		1.645E-19	5.148E-28

SOURCE REGION CALCULATIONAL RESULTS

RECEIVER POINT DET.PT. ONE
COORDINATES R..... 1.22000E+02
Z..... 1.52000E+01

PHI.... 0.

NEUT. 1 NEUT. 2

1 0°	2.756E-17	1.399E-12
2 0°	8.964E-16	4.621E-11
3 0°	2.454E-14	1.378E-09
4 0°	8.225E-14	4.810E-09
5 0°	3.770E-13	2.332E-08
6 0°	9.898E-13	6.643E-08
7 0°	1.095E-12	8.619E-08
8 0°	2.225E-12	2.060E-07
9 0°	5.291E-12	6.068E-07
10 0°	3.3A6E-12	7.696E-07
	-----	-----
	1.347E-11	1.745E-06

CALCULATED RESULTS FOR SOURCE REGION 1

SOURCE REGION CALCULATIONAL RESULTS

RECEIVER POINT RET.PT. TWO
 COORDINATES R..... 1.22000E+02
 Z..... 9.16000E+01
 PHI.... 0.

	GAM RESP 1	GAM RESP 2
1 8.50	2.691E-20	1.238E-20
2 7.25	1.123E-19	4.887E-20
3 6.50	8.740E-20	3.582E-20
4 5.50	1.468E-19	5.702E-20
5 4.50	9.721E-20	3.351E-20
6 3.50	1.885E-19	5.885E-20
7 2.80	1.240E-19	3.650E-20
8 2.50	1.069E-18	3.073E-27
9 2.00	1.717E-18	4.638E-27

10	1.50	5.321E-19	1.395E-27
11	1.00	3.740E-18	9.802E-27
12	.70	4.642E-19	1.256E-27
13	.30	5.243E-20	1.721E-28
		-----	-----
		6.324E-18	2.317E-26

SOURCE REGION CALCULATIONAL RESULTS

RECEIVER POINT DET.PT. TWO
 COORDINATES R..... 1.22000E+02
 Z..... 9.1600E+01
 PHI.... 0.

GAM RESP 1 GAM RESP 2

1	8.50	4.003E-21	1.841E-29
2	7.25	1.515E-20	6.592E-29
3	6.50	1.050E-20	4.302E-29
4	5.50	1.531E-20	5.948E-29
5	4.50	8.481E-21	2.924E-29
6	3.50	1.293E-20	4.036E-29
7	2.80	6.795E-21	1.982E-29
8	2.50	5.113E-20	1.470E-28
9	2.00	1.019E-20	2.805E-29
10	1.50	1.295E-20	3.395E-29
11	1.00	4.590E-20	1.203E-28
12	.70	2.817E-21	7.623E-30
13	.30	1.001E-22	3.285E-31
		-----	-----
		1.964E-19	6.115E-28

SOURCE REGION CALCULATIONAL RESULTS

RECEIVER POINT DET.PT. TWO
 COORDINATES R..... 1.22000E+02



Z..... 9.16000E+01
PHI.... 0.

NEUT. 1 NEUT. 2

1 0.	3.413E-17	1.732E-12
2 0.	1.059E-15	5.457E-11
3 0.	2.878E-14	1.617E-09
4 0.	9.646E-14	5.641E-09
5 0.	4.455E-13	2.750E-08
6 0.	1.176E-12	7.893E-08
7 0.	1.315E-12	1.035E-07
8 0.	2.681E-12	2.493E-07
9 0.	6.498E-12	7.349E-07
10 0.	4.114E-12	9.351E-07
	-----	-----
	1.627E-11	2.116E-06

CALCULATED RESULTS FOR SOURCE REGION 1

SOURCE REGION CALCULATIONAL RESULTS

RECEIVER POINT DET.PT.THREE
COORDINATES R..... 1.22000E+02
Z..... 1.68000E+02
PHI.... 0.

GAM RESP 1 GAM RESP 2

1 8.50	7.343E-21	3.378E-29
2 7.25	3.022E-20	1.315E-28
3 6.50	2.329E-20	9.545E-29
4 5.50	3.845E-20	1.493E-28
5 4.50	2.480E-20	8.548E-29
6 3.50	4.587E-20	1.432E-28
7 2.80	2.854E-20	8.401E-29
8 2.50	2.386E-19	6.856E-28



Astronuclear
Laboratory

9	2.00	3.598E-19	9.716E-28
10	1.50	1.004E-19	2.633E-28
11	1.00	5.893E-19	1.544E-27
12	.70	5.959E-20	1.613E-28
13	.30	2.040E-21	6.697E-30
		1.548E-18	4.356E-27

SOURCE REGION CALCULATIONAL RESULTS

RECEIVER POINT DET.PT.THREE
COORDINATES R..... 1.22000E+02
Z..... 1.68000E+02
PHI.... 0.

GAM RESP 1 GAM RESP 2

1	0.50	1.092E-21	5.023E-30
2	7.25	4.077E-21	1.774E-29
3	6.50	2.798E-21	1.147E-29
4	5.50	4.011E-21	1.558E-29
5	4.50	2.163E-21	7.458E-30
6	3.50	3.146E-21	9.820E-30
7	2.50	1.550E-21	4.563E-30
8	2.50	1.141E-20	3.278E-29
9	2.00	2.175E-21	5.075E-30
10	1.50	2.444E-21	6.407E-30
11	1.00	7.232E-21	1.895E-29
12	.70	3.616E-22	9.786E-31
13	.30	3.893E-24	1.278E-32
		4.0246E-20	1.367E-28

SOURCE REGION CALCULATIONAL RESULTS

RECEIVER POINT DET.PT.THREE

COORDINATES R..... 1.22000E+02
 Z..... 1.68000E+02
 PHI.... 0.

NEUT. 1 NEUT. 2

	NEUT. 1	NEUT. 2
1	0°	7.487E-10
2	0°	3.197E-16
3	0°	8.682E-15
4	0°	2.909E-14
5	0°	1.275E-13
6	0°	3.032E-13
7	0°	3.072E-13
8	0°	6.062E-13
9	0°	1.294E-12
10	0°	8.097E-13
		3.486E-12
		4.432E-07

CALCULATED RESULTS FOR SOURCE REGION 1

SOURCE REGION CALCULATIONAL RESULTS

RECEIVER POINT DET.PT. FOUR
 COORDINATES R..... 9.16000E+01
 Z..... 2.28000E+02
 PHI.... 0.

GAM RESP 1 GAM RESP 2

	GAM RESP 1	GAM RESP 2
1	8.50	7.586E-22
2	7.25	3.139E-21
3	6.50	2.407E-21
4	5.50	3.886E-21
5	4.50	2.372E-21
6	3.50	3.906E-21
7	2.60	2.072E-21
		6.097E-30

6	2.50	1.561E-20	4.487E-29
9	2.00	1.693E-20	5.084E-29
10	1.50	3.596E-21	9.428E-30
11	1.00	1.095E-20	2.870E-29
12	.70	5.510E-22	1.491E-30
13	.30	9.759E-24	3.204E-32
		6.84E-20	2.039E-28

SOURCE REGION CALCULATIONAL RESULTS

RECEIVER POINT DET.PT. FOUR
 COORDINATES R..... 9.16000E+01
 Z..... 2.28000E+02
 PH1..... 0.

GAM RESP 1 GAM RESP 2

1	0.50	1.128E-22	5.190E-31
2	7.25	4.235E-22	1.843E-30
3	6.50	2.891E-22	1.185E-30
4	5.50	4.053E-22	1.574E-30
5	4.50	2.070E-22	7.135E-31
6	3.50	2.679E-22	8.362E-31
7	2.80	1.125E-22	3.311E-31
8	2.50	7.465E-22	2.145E-30
9	2.00	1.138E-22	3.074E-31
10	1.50	8.751E-23	2.294E-31
11	1.00	1.344E-22	3.522E-31
12	.70	3.343E-24	9.049E-33
13	.30	1.062E-26	6.115E-35
		2.934E-21	1.005E-29

SOURCE REGION CALCULATIONAL RESULTS

COORDINATES R..... 9.16000E+01
 Z..... 2.28000E+02
 PHI... 0.

NEUT. 1 NEUT. 2

1 0°	1.398E-18	7.098E-14
12 0°	1.089E-16	5.616E-12
3 0°	2.961E-15	1.643E-10
4 0°	1.019E-14	5.957E-10
5 0°	4.213E-14	2.601E-09
6 0°	8.970E-14	6.020E-09
7 0°	8.360E-14	6.582E-09
8 0°	1.563E-13	1.447E-08
9 0°	3.323E-13	3.811E-08
10 0°	2.036E-13	4.628E-08
	-----	-----
9.2 9E-13	1.148E-07	

CALCULATED RESULTS FOR SOURCE REGION 1

SOURCE REGION CALCULATIONAL RESULTS

RECEIVER POINT DET.PT. FIVE
 COORDINATES R..... 3.05000E+01
 Z..... 2.28000E+02
 PHI... 0.

GAM RESP 1 GAM RESP 2

1 8.50	1.846E-21	8.493E-30
2 7.25	7.787E-21	3.389E-29
3 6.50	6.050E-21	2.479E-29
4 5.50	9.986E-21	3.879E-29
5 4.50	6.299E-21	2.172E-29
6 3.50	1.098E-20	3.428E-29
7 2.80	6.218E-21	1.630E-29

6	2.50	4.862E-20	1.397E-20
9	2.00	6.310E-20	1.704E-20
10	1.50	1.361E-20	3.567E-20
11	1.00	5.064E-20	1.327E-20
12	.70	3.135E-21	8.486E-30
13	.30	1.099E-22	3.610E-31
		-----	-----
		2.284E-19	6.677E-20

SOURCE REGION CALCULATIONAL RESULTS

RECEIVER POINT DET.PT. FIVE
 COORDINATES R..... 3.05000E+01
 Z..... 2.28000E+02
 PHI.... 0.

	GAM RESP 1	GAM RESP 2
1	8.50	2.746E-22
2	7.25	1.051E-21
3	6.50	7.268E-22
4	5.50	1.042E-21
5	4.50	5.495E-22
6	3.50	7.531E-22
7	2.80	3.377E-22
8	2.50	2.325E-21
9	2.00	3.816E-22
10	1.50	3.311E-22
11	1.00	6.215E-22
12	.70	1.903E-23
13	.30	2.098E-25

		8.4 2E=21

SOURCE REGION CALCULATIONAL RESULTS

COORDINATES R..... 3.05000E+01
 Z..... 2.28000E+02
 PHI.... 0.

NEUT. 1 NFUT. 2

1	0°	5.248E-18	2.664E-13
2	0°	2.941E-16	1.516E-11
3	0°	8.019E-15	4.505E-10
4	0°	2.669E-14	1.561E-09
5	0°	1.152E-13	7.111E-09
6	0°	2.629E-13	1.0764E-08
7	0°	2.574E-13	2.027E-08
8	0°	4.941E-13	4.612E-08
9	0°	1.061E-12	1.217E-07
10	0°	6.594E-13	1.499E-07
		-----	-----
		2.889E-12	3.647E-07

FOR SOURCE REGION 1 AND DETECTOR POINT 5 THERE HAS BEEN 5928 AND 243 PATH LENGTH CALCULATIONS IN EXCESS OF 0. MEAN FREE PATHS(GAMMA RAY) AND 120.0 GRAMS/CM**2(NEUTRON), RESPECTIVELY

INTERMEDIATE SUMMARY RESULTS OVER A SUBSET OF SOURCE REGIONS

SUBTOTAL OVER SELECTED SOURCE

RECEIVER POINT DET.PT. ONE
 COORDINATES R..... 1.22000E+02
 Z..... 1.52000E+01
 PHI.... 0.

GAM RESP 1 GAM RESP 2

1	8.50	2.297E-20	1.056E-20
2	7.25	9.569E-20	4.165E-20
3	6.50	7.442E-20	3.050E-20
4	5.50	1.248E-19	4.648E-20

5	4.50	8.248E-20
6	3.50	1.594E-19
7	2.80	1.044E-19
8	2.50	8.978E-19
9	2.00	1.435E-18
10	1.50	4.415E-19
11	1.00	3.068E-18
12	.70	3.770E-19
13	.30	4.168E-20
		6.926E-18

SUBTOTAL OVER SELECTED SOURCE

RECEIVER POINT NET.PT. ONF
 COORDINATES R..... 1.22000E+02
 Z..... 1.52000E+01
 PHI.... 0.

GAM RESP 1 GAM RESP 2

1	8.50	3.416E-21
2	7.25	1.291E-20
3	6.50	6.940E-21
4	5.50	1.302E-20
5	4.50	7.196E-21
6	3.50	1.093E-20
7	2.80	5.668E-21
8	2.50	4.293E-20
9	2.00	8.680E-21
10	1.50	1.074E-20
11	1.00	3.766E-20
12	.70	2.288E-21
13	.30	7.955E-23
		1.645E-19

SUBTOTAL OVER SELECTED SOURCE



RECEIVER POINT DET.PT. ONE
 COORDINATES R..... 1.22000E+02
 Z..... 1.92000E+01
 PHI.... 0.

NEUT. 1 NEUT. 2

1	0:	2.756E-17	1.399E-12
2	0:	8.964E-16	4.621E-11
3	0:	2.454E-14	1.378E-09
4	0:	8.225E-14	4.810E-09
5	0:	3.778E-13	2.332E-08
6	0:	9.898E-13	6.643E-08
7	0:	1.095E-12	8.619E-08
8	0:	2.225E-12	2.060E-07
9	0:	5.291E-12	6.048E-07
10	0:	3.386E-12	7.696E-07
		-----	-----
		1.347E-11	1.765E-06

INTERMEDIATE SUMMARY RESULTS OVER A SUBSET OF SOURCE REGIONS

SUBTOTAL OVER SELECTED SOURCE

RECEIVER POINT DET.PT. TWO
 COORDINATES R..... 1.22000E+02
 Z..... 9.16000E+01
 PHI.... 0.

GAM RESP 1 GAM RESP 2

1	8.50	2.691E-20	1.238E-28
2	7.25	1.123E-19	4.887E-28
3	6.50	6.740E-20	3.582E-28



Astronuclear
Laboratory

4	5.50	1.468E-19
5	4.50	9.721E-20
6	3.50	1.0885E-19
7	2.80	1.0240E-19
8	2.50	1.069E-18
9	2.00	1.717E-18
10	1.50	5.321E-19
11	1.00	3.740E-18
12	.70	4.642E-19
13	.30	5.243E-20
		A. 3288E-18

SUBTOTAL OVER SELECTED SOURCE

RECEIVER POINT RET.PT. TWO
COORDINATES R..... 1.22000E+02
Z..... 9.16000E+01
PHI.... 0.

GAM RESP 1 GAM RESP 2

1	8.50	4.003E-21	1.841E-29
2	7.25	1.515E-20	6.592E-29
3	6.50	1.050E-20	4.302E-29
4	5.50	1.531E-20	5.948E-29
5	4.50	6.481E-21	2.924E-29
6	3.50	1.293E-20	4.036E-29
7	2.80	6.735E-21	1.982E-29
8	2.50	5.113E-20	1.470E-28
9	2.00	1.039E-20	2.805E-29
10	1.50	1.295E-20	3.395E-29
11	1.00	4.590E-20	1.203E-28
12	.70	2.817E-21	7.623E-30
13	.30	1.001E-22	3.295E-31
			1.964E-19

SUBTOTAL OVER SELECTED SOURCE



RECEIVER POINT DET.PT. TWO
 COORDINATES R..... 1.22000E+02
 Z..... 9.16000E+01
 PHI.... 0.

NEUT. 1 NEUT. 2

1	0.	3.413E+17	1.732E+12
2	0.	1.059E+15	5.457E+11
3	0.	2.878E+14	1.617E+09
4	0.	9.646E+14	5.641E-09
5	0.	4.455E+13	2.750E+08
6	0.	1.176E+12	7.893E+08
7	0.	1.315E+12	1.035E+07
8	0.	2.681E+12	2.483E+07
9	0.	6.408E+12	7.349E+07
10	0.	4.114E+12	9.351E+07
		-----	-----
		1.627E+11	2.136E+06

INTERMEDIATE SUMMARY RESULTS OVER A SUBSET OF SOURCE REGIONS

SUBTOTAL OVER SELECTED SOURCE

RECEIVER POINT DET.PT. THREE
 COORDINATES R..... 1.22000E+02
 Z..... 1.68000E+02
 PHI.... 0.

GAM RESP 1 GAM RESP 2

1	8.50	7.343E+21	3.378E+29
2	7.25	3.022E+20	1.315E+28

3	6.50	2.329E-20	9.545E-20
4	5.50	3.845E-20	1.064E-20
5	4.50	2.40E-20	6.2E-20
6	3.50	4.587E-20	1.4512E-20
7	2.80	2.054E-20	6.04E-20
8	2.50	2.0366E-19	6.0856E-19
9	2.00	3.598E-19	9.716E-20
10	1.50	1.004E-19	2.633E-20
11	1.00	5.083E-19	1.0544E-20
12	.70	5.959E-20	1.613E-20
13	.30	2.040E-21	6.697E-30
		1.548E-18	4.356E-21

SUBTOTAL OVER SELECTED SOURCE

RECEIVER POINT DET.PT.THREE
 COORDINATES R..... 1.22000E+02
 Z..... 1.68000E+02
 PHI.... 0.

GAM RESP 1 GAM RESP 2

1	8.50	1.092E-21	5.023E-30
2	7.25	4.077E-21	1.774E-29
3	6.50	2.798E-21	1.147E-29
4	5.50	4.011E-21	1.558E-29
5	4.50	2.193E-21	7.458E-30
6	3.50	3.146E-21	9.820E-30
7	2.80	1.550E-21	4.563E-30
8	2.50	1.141E-20	3.278E-29
9	2.00	2.175E-21	5.875E-30
10	1.50	2.444E-21	6.407E-30
11	1.00	7.232E-21	1.0895E-29
12	.70	3.619E-22	9.788E-31
13	.30	3.089E-24	1.2748E-32
		6.246E-20	1.367E-28

SUBTOTAL OVER SELECTED SOURCE

RECEIVER POINT DET.PT. THREE
 COORDINATES R..... 1.22000E+02
 Z..... 1.68000E+02
 PHI.... 0.

NFUT. 1

1	0°	7.487E-18	3.81E-13
2	0°	3.197E-16	1.648E-11
3	0°	8.682E-15	4.877E-10
4	0°	2.909E-14	1.701E-09
5	0°	1.275E-13	7.873E-09
6	0°	3.032E-13	2.035E-08
7	0°	3.072E-13	2.419E-08
8	0°	6.062E-13	5.613E-08
9	0°	1.294E-12	1.484E-07
10	0°	8.097E-13	1.840E-07
-----		-----	-----
		3.486E-12	4.432E-07

INTERMEDIATE SUMMARY RESULTS OVER A SUBSET OF SOURCE REGIONS

SUBTOTAL OVER SELECTED SOURCE

RECEIVER POINT DET.PT. FOUR
 COORDINATES R..... 9.16000E+01
 Z..... 2.28000E+02
 PHI.... 0.

GAM RESP 1 GAM RESP 2

1 0.50 7.586E-22 3.490E-30



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2	7.25	3.139E-21	1.366E-29
3	6.50	2.407E-21	9.862E-30
4	5.50	3.886E-21	1.509E-29
5	4.50	2.372E-21	9.178E-30
6	3.50	3.986E-21	1.219E-29
7	2.80	2.072E-21	6.097E-30
8	2.50	1.561E-20	4.487E-29
9	2.00	1.883E-20	5.084E-29
10	1.50	3.596E-21	9.428E-30
11	1.00	1.095E-20	2.870E-29
12	.70	5.510E-22	1.491E-30
13	.30	9.759E-24	3.204E-32
		-----	-----
		6.818E-20	2.039E-28

SUBTOTAL OVER SELECTED SOURCE

RECEIVER POINT RET.PT. FOUR
COORDINATES R..... 9.16000E+01
Z..... 2.28000E+02
PHI.... 0.

	GAM RESP 1	GAM RESP 2
1	8.50	1.128E-22
2	7.25	4.235E-22
3	6.50	2.891E-22
4	5.50	4.053E-22
5	4.50	2.070E-22
6	3.50	2.679E-22
7	2.80	1.125E-22
8	2.50	7.465E-22
9	2.00	1.13AE-22
10	1.50	8.751E-23
11	1.00	1.344E-22
12	.70	3.343E-24
13	.30	1.862E-26

		2.94E-21
		1.055E-29



Astronuclear
Laboratory

SUBTOTAL OVER SELECTED SOURCE

RECEIVER POINT DET.PT. FOUR
COORDINATES R..... 9.1600E+01
Z..... 2.2800E+02
PHI.... 0.

NEUT. 1 NEUT. 2

1	0.	1.398E+18	7.098E+14
2	0.	1.089E+16	5.616E+12
3	0.	2.961E+15	1.663E+10
4	0.	1.019E+14	5.957E+10
5	0.	4.213E+14	2.601E+09
6	0.	8.970E+14	6.020E+09
7	0.	8.360E+14	6.582E+09
8	0.	1.563E+13	1.447E+08
9	0.	3.323E+13	3.811E+08
10	0.	2.036E+13	4.628E+08
		-----	-----
	9.299E+13	1.148E+07	

INTERMEDIATE SUMMARY RESULTS OVER A SUBSET OF SOURCE REGIONS

SUBTOTAL OVER SELECTED SOURCE

RECEIVER POINT DET.PT. FIVE
COORDINATES R..... 3.0500E+01
Z..... 2.2800E+02
PHI... 0.

GAM RESP 1 GAM RESP 2

1	8.50	1.846E-21	8.493E-30
2	7.25	7.787E-21	3.389E-29
3	6.50	6.050E-21	2.479E-29
4	5.50	9.986E-21	3.879E-29
5	4.50	6.299E-21	2.172E-29
6	3.50	1.098E-20	3.428E-29
7	2.80	6.218E-21	1.830E-29
8	2.50	4.862E-20	1.397E-28
9	2.00	6.310E-20	1.704E-28
10	1.50	1.361E-20	3.567E-29
11	1.00	5.064E-20	1.327E-28
12	.70	3.135E-21	8.486E-30
13	.30	1.069E-22	3.610E-31
		2.284E-19	6.677E-28

SUMTOTAL OVER SELECTED SOURCE

RECEIVER POINT NET.PT. FIVE
 COORDINATES R..... 3.05000E+01
 Z..... 2.28000F+02
 PHI.... 0.

	GAM RESP 1	GAM RESP 2
1	8.50	2.746E-22
2	7.25	1.051E-21
3	6.50	7.268E-22
4	5.50	1.042E-21
5	4.50	5.495E-22
6	3.50	7.531E-22
7	2.80	3.377E-22
8	2.50	2.325E-21
9	2.00	3.816E-22
10	1.50	3.311E-22
11	1.00	6.215E-22
12	.70	1.903E-23
13	.30	2.098E-25
		2.284E-19
		8.42E-21
		2.876E-29

SUBTOTAL OVER SELECTED SOURCE

RECEIVER POINT NEUT.PT. FIVE
 COORDINATES R..... 3.05000E+01
 Z..... 2.28000E+02
 PHI.... 0.

NEUT. 1 NEUT. 2

1	0°	5.248E-18	2.664E-13
2	0°	2.941E-16	1.516E-11
3	0°	8.019E-15	4.505E-10
4	0°	2.669E-14	1.561E-09
5	0°	1.152E-13	7.111E-09
6	0°	2.629E-13	1.744E-08
7	0°	2.574E-13	2.027E-08
8	0°	4.941E-13	4.612E-08
9	0°	1.061E-12	1.217E-07
10	0°	6.594E-13	1.499E-07
-----		-----	
		2.889E-12	3.647E-07

CUMULATIVE SUMMARY RESULTS OVER ALL SUBSETS OF SOURCE REGIONS
SUMMATION OVER ALL SOURCE REGIONS

RECEIVER POINT
 COORDINATES R..... 1.22000E+02
 Z..... 1.52000E+01
 PHI.... 0.

GAM RESP 1 GAM RESP 2



Astronuclear
Laboratory

1	8.50	2.297E-20
2	7.25	9.569E-20
3	6.50	7.442E-20
4	5.50	1.248E-19
5	4.50	8.248E-20
6	3.50	1.594E-19
7	2.50	1.044E-19
8	2.50	8.978E-19
9	2.00	1.435E-18
10	1.50	4.415E-19
11	1.00	3.068E-18
12	.70	3.770E-19
13	.30	4.160E-20

		6.926E-18
		1.922E-26

SUMMATION OVER ALL SOURCE REGIONS

RECEIVER POINT	
COORDINATES R.....	1.22000E+02
Z.....	0.
PHI....	0.

GAM RESP 1 GAM RESP 2

1	8.50	3.416E-21	1.571E-29
2	7.25	1.291E-20	5.619E-29
3	6.50	8.940E-21	3.693E-29
4	5.50	1.302E-20	5.051E-29
5	4.50	7.196E-21	2.491E-29
6	3.50	1.093E-20	3.412E-29
7	2.50	5.668E-21	1.664E-29
8	2.50	4.293E-20	1.234E-28
9	2.00	8.680E-21	2.344E-28
10	1.50	1.074E-20	2.817E-29
11	1.00	3.766E-20	9.870E-29
12	.70	2.288E-21	6.192E-30
13	.30	7.955E-23	2.612E-31
		-----	-----
		1.645E+19	5.148E-28

SUMMATION OVER ALL SOURCE REGIONS

RECEIVER POINT
 COORDINATES R.....
 Z..... 1.2200E+02
 PHI.... 1.5200E+01
 0.

NEUT. 1 NEUT. 2

1	0°	2.756E-17	1.399E-12
2	0°	8.964E-16	4.621E-11
3	0°	2.454E-14	1.378E-10
4	0°	8.225E-14	4.810E-10
5	0°	3.078E-13	2.332E-09
6	0°	9.988E-13	6.643E-08
7	0°	1.095E-12	8.619E-08
8	0°	2.025E-12	2.060E-07
9	0°	5.291E-12	6.048E-07
10	0°	3.346E-12	7.696E-07
		-----	-----
		1.347E-11	1.745E-06

CUMULATIVE SUMMARY RESULTS OVER ALL SUBSETS OF SOURCE REGIONS
SUMMATION OVER ALL SOURCE REGIONS

RECEIVER POINT
 COORDINATES R.....
 Z..... 1.2200E+02
 PHI.... 9.1600E+01
 0.

GAM RESP 1 GAM RESP 2

1	8.50	2.691E-20
2	7.25	1.123E-19
3	6.50	8.740E-20
4	5.50	1.468E-19
5	4.50	9.721E-20
6	3.50	1.865E-19
7	2.50	1.240E-19
8	2.00	1.069E-18
9	1.50	1.717E-18
10	1.00	5.321E-19
11	1.00	3.740E-18
12	.70	4.662E-19
13	.30	5.243E-20
		8.358E-18
		2.317E-26

SUMMATION OVER ALL SOURCE REGIONS

RECEIVER POINT COORDINATES R.....	Z.....	PHI....
		1.22000E+0?
		9.16000E+01
		0.

GAM RESP 1 GAM RESP ?

1	8.50	4.003E-21
2	7.25	1.515E-20
3	6.50	1.050E-20
4	5.50	1.531E-20
5	4.50	8.491E-21
6	3.50	1.293E-20
7	2.50	6.715E-21
8	2.00	5.113E-20
9	1.50	1.039E-20
10	1.00	1.295E-20
11	1.00	4.590E-20
12	.70	2.817E-21
13	.30	1.001E-22
		1.964E-19
		6.115E-26

SUMMATION OVER ALL SOURCE REGIONS

RECEIVER POINT	R.....	1.22000E+02
COORDINATES	Z.....	9.16000E+01
	PHI....	0.

NEUT. 1 NEUT. 2

1	0.	3.413E-17
2	0.	1.059E-15
3	0.	2.070E-14
4	0.	9.646E-14
5	0.	4.455E-13
6	0.	1.176E-12
7	0.	1.315E-12
8	0.	2.691E-12
9	0.	6.408E-12
10	0.	4.114E-12

		1.627E-11

2.136E-06

CUMULATIVE SUMMARY RESULTS OVER ALL SUBSETS OF SOURCE REGIONS

SUMMATION OVER ALL SOURCE REGIONS

RECEIVER POINT	R.....	1.22000E+02
COORDINATES	Z.....	9.16000E+01
	PHI....	0.

GAM RESP 1 GAM RESP 2



1	8.50	7.343E-21
2	7.25	3.022E-20
3	6.50	2.329E-20
4	5.50	3.845E-20
5	4.50	2.480E-20
6	3.50	4.587E-20
7	2.80	2.854E-20
8	2.50	2.386E-19
9	2.00	3.598E-19
10	1.50	1.004E-19
11	1.00	5.893E-19
12	.70	5.959E-20
13	.30	2.040E-21
		1.548E-18

SUMMATION OVER ALL SOURCE REGIONS

RECEIVER POINT COORDINATES R.....	1.22000E+02
Z.....	1.68000E+02
PHI....	0.

GAM RESP 1 GAM RESP 2

1	8.50	1.092E-21	5.023E-30
2	7.25	4.077E-21	1.074E-29
3	6.50	2.798E-21	1.147E-29
4	5.50	4.011E-21	1.558E-29
5	4.50	2.163E-21	7.458E-30
6	3.50	3.146E-21	9.820E-30
7	2.80	1.550E-21	4.563E-30
8	2.50	1.0141E-20	3.278E-29
9	2.00	2.175E-21	5.875E-30
10	1.50	2.444E-21	6.407E-30
11	1.00	7.232E-21	1.895E-29
12	.70	3.616E-22	9.796E-31
13	.30	3.893E-24	1.278E-32
		4.246E-20	1.367E-28

SUMMATION OVER ALL SOURCE REGIONS

RECEIVER POINT	
COORDINATES R.....	1.022000E+02
Z.....	1.680000E+02
PHI....	0.

NEUT. 1 NEUT. 2

1	0.	7.487E-18	3.801E-13
2	0.	3.197E-16	1.648E-11
3	0.	8.682E-15	4.877E-10
4	0.	2.909E-14	1.701E-09
5	0.	1.275E-13	7.873E-09
6	0.	3.032E-13	2.035E-08
7	0.	3.072E-13	2.419E-08
8	0.	6.062E-13	5.613E-08
9	0.	1.294E-12	1.484E-07
10	0.	6.097E-13	1.840E-07
		-----	-----
		3.486E-12	4.432E-07

CUMULATIVE SUMMARY RESULTS OVER ALL SUBSETS OF SOURCE REGIONS

SUMMATION OVER ALL SOURCE REGIONS

RECEIVER POINT	
COORDINATES R.....	9.16000E+01
Z.....	2.28000E+02
PHI....	0.

GAM RESP 1 GAM RESP 2



1	8.50	7.986E-22
2	7.25	3.139E-21
3	6.50	2.407E-21
4	5.50	3.088E-21
5	4.50	2.372E-21
6	3.50	3.090E-21
7	2.50	2.072E-21
8	2.50	1.561E-20
9	2.00	1.683E-20
10	1.50	3.596E-21
11	1.00	1.095E-20
12	.70	5.510E-22
13	.30	9.759E-24

		6.8E-20
		2.039E-28

SUMMATION OVER ALL SOURCE REGIONS

RECEIVER POINT	R.....	9.16000E+01
COORDINATES	Z.....	2.28000E+02
	PHI....	0.

GAM RESP 1 GAM RESP 2

1	8.50	1.120E-22	5.190E-31
2	7.25	4.235E-22	1.843E-30
3	6.50	2.691E-22	1.165E-30
4	5.50	4.053E-22	1.574E-30
5	4.50	2.070E-22	7.135E-31
6	3.50	2.679E-22	6.362E-31
7	2.50	1.125E-22	3.311E-31
8	2.50	7.465E-22	2.145E-30
9	2.00	1.130E-22	3.074E-31
10	1.50	8.751E-23	2.294E-31
11	1.00	1.344E-22	3.522E-31
12	.70	3.343E-24	9.049E-33
13	.30	1.862E-26	6.115E-35
		-----	-----
		2.914E-21	1.005E-29



SUMMATION OVER ALL SOURCE REGIONS

RECEIVER POINT
COORDINATES R.....
Z.....
PHI....
0.

9.16000E+01
2.26000E+02

NEUT. 1 NEUT. 2

1 0.	1.398E-18	7.098E-14
2 0.	1.049E-16	5.616E-12
3 0.	2.961E-15	1.663E-10
4 0.	1.019E-14	5.957E-10
5 0.	4.213E-14	2.601E-09
6 0.	8.970E-14	6.020E-09
7 0.	8.360E-14	6.582E-09
8 0.	1.563E-13	1.447E-08
9 0.	3.323E-13	3.811E-08
10 0.	2.036E-13	4.628E-08
	-----	-----
	9.219E+13	1.144E+07

CUMULATIVE SUMMARY RESULTS OVER ALL SUBSETS OF SOURCE REGIONS

SUMMATION OVER ALL SOURCE REGIONS

RECEIVER POINT
COORDINATES R.....
Z.....
PHI....
0.

3.05000E+01
2.28000E+02
0.

GAM RESP 1 GAM RESP 2



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1	8.50	1.846E+21
2	7.25	7.787E+21
3	6.50	6.050E+21
4	5.50	9.986E+21
5	4.50	6.299E+21
6	3.50	1.098E+20
7	2.50	6.218E+21
8	2.50	4.862E+20
9	2.00	6.310E+20
10	1.50	1.361E+20
11	1.00	5.064E+20
12	.70	3.135E+21
13	.30	1.099E+22
		2.284E+19

SUMMATION OVER ALL SOURCE REGIONS

RECEIVER POINT
COORDINATES R.....
Z.....
PHI....

3.05000E+01
2.26000E+02
0.

GAM RESP 1 GAM RESP 2

1	8.50	2.746E+22	1.263E+30
2	7.25	1.051E+21	4.572E+30
3	6.50	7.268E+22	2.978E+30
4	5.50	1.042E+21	4.046E+30
5	4.50	5.495E+22	1.895E+30
6	3.50	7.531E+22	2.351E+30
7	2.50	3.377E+22	9.939E+31
8	2.50	2.325E+21	6.681E+30
9	2.00	3.816E+22	1.030E+30
10	1.50	3.311E+22	8.681E+31
11	1.00	6.215E+22	1.629E+30
12	.70	1.903E+23	5.149E+32
13	.30	2.098E+25	6.889E+34
		-----	-----
		-----	2.036E+29
		-----	8.412E+21

SUMMATION OVER ALL SOURCE REGIONS

PRECIPITATION POINT
 COORDINATES R.....
 Z.....
 PHI....
 0.

NFUT. 1

NFUT. 2

1	0°	5.248E-18	2.664E-13
2	0°	2.941E-16	1.516E-11
3	0°	8.019E-15	4.505E-10
4	0°	2.669E-14	1.561E-09
5	0°	1.152E-13	7.111E-09
6	0°	2.629E-13	1.764E-08
7	0°	2.574E-13	2.027E-08
8	0°	4.991E-13	4.612E-08
9	0°	1.061E-12	1.217E-07
10	0°	6.594E-13	1.499E-07
		2.889E-12	3.647E-07

ERROR IN INPUT DATA CARD AS FOLLOWS

-n-n -0



SECTION

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4. Capo, M. A., and Woodsum, H. C., Second Status Report of WANL Shielding Standard Design Method, Westinghouse Astronuclear Laboratory, WANL-TME-1466, March, 1967.
5. Capo, M. A., Gamma Ray Absorption Coefficients for Elements and Mixture, General Electric Co., ANPD, XDC 59-10-19, September 28, 1959.
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8. Storm, E., et. al., Gamma Ray Absorption Coefficients for Elements 1 Through 100 Derived from Theoretical Values of the National Bureau of Standards, Los Alamos Scientific Laboratory, LA-2237, November, 1958.
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APPENDIX

Presented here is a listing of the FORTRAN IV Source Program for the KAP-V code for use on the IBM 7094 computer.



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SIBSYS
$JOB    IBJOB
$EXECUTE GO, SOURCE, MAP, F1OCS, FILES
$IBJOB   KAPV      NOLIST, DECK, NODD, M94/2, XR7
$IBFTC  KAPV      COMMON AI(550), NI(1700), FI(5660)
EQUIVALENCE (AI(1),TITLE) 1
EQUIVALENCE (NI(1),NGG) 1
EQUIVALENCE (NI(4),NCOMP) 1
EQUIVALENCE (NI(7),NREG) 1
EQUIVALENCE (NI(10),NRSPA) 1
EQUIVALENCE (NI(17),NOVG) 1
EQUIVALENCE (NI(20),IGAM) 1
EQUIVALENCE (NI(25),IZSD) 1
EQUIVALENCE (NI(28),ISZC) 1
EQUIVALENCE (NI(31),ISUM) 1
EQUIVALENCE (NI(35),KORD) 1
EQUIVALENCE (NI(50),LSO) 1
EQUIVALENCE (NI(75),LSIT) 1
EQUIVALENCE (NI(100),NEQBD) 1
EQUIVALENCE (NI(400),NBILD) 1
EQUIVALENCE (FI(1),ASOI) 1
EQUIVALENCE (FI(7),RS) 1
EQUIVALENCE (FI(470),FSI) 1
EQUIVALENCE (FI(1460),ABD) 1
EQUIVALENCE (FI(1760),XOBBD) 1
EQUIVALENCE (FI(2060),DBD) 1
EQUIVALENCE (FI(2460),COMP) 1
EQUIVALENCE (FI(3490),XSECN) 1
EQUIVALENCE (FI(3558),AWSOUR) 1
EQUIVALENCE (FI(3560),ENG) 1
EQUIVALENCE (FI(4310),SMFP) 1
EQUIVALENCE (FI(4312),EPSLN) 1
EQUIVALENCE (FI(4320),COM) 1
EQUIVALENCE (FI(4750),RSPG) 1
EQUIVALENCE (FI(5360),RCORD) 1
EQUIVALENCE (FI(5533),RSIT) 1
EQUIVALENCE (NI(120),INEUT(3)) 1
DIMENSION NBNDZN(100),NEQBD(100),NCMPZN(100)
DIMENSION NTRYZN(6,100)
DIMENSION ASOI(2),XI(2),ETA(2),RS(2),ZS(2),PHI(21,20)
DIMENSION FSI(21,22,2),GSOUR(30),NSOUR(30)
DIMENSION RSIT(21),ZSIT(21),FSIT(21,2,2)
DIMENSION ASO(2),RSI(20),ZSI(20),PHI(120,20),FS(20,22,2)
DIMENSION ABD(100),BBD(100),CBD(100),BBD(100),YBBD(100)
DIMENSION ZOBBD(100),COMP(20,50),XYZ(3,100)
DIMENSION ANEUT(31),AGAM(30),UAGAM(30)

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DIMENSION ENN(30),XSECN(20,3),ALFA(9),ENG(30),XSECG(30,20) KAP 43
DIMENSION BILD(4,30,1),BKP(4),COM(5,30,2),CON(5,5,4),XLAM(30) KAP 44
DIMENSION RSPG(30,10),RSPN(30,10),RCORD(3,50),RSPA(1,10) KAP 45
DIMENSION FLUX(151),FLUXS(151,50),XSECS(30),FSP(2),SVOL(30) KAP 46
DIMENSION FSR(2) KAP 47
DIMENSION VOL(30),SVUC(30),CORR(30) KAP 48
DIMENSION AX(151),NX(201),FX(5),R(6) KAP 49
DIMENSION SSOT(3),TITLE(5501),SREG(1001),ZAT(20) KAP 50
REAL NSOUR KAP 51
DATA BLANK / 6H /
*****KAP-IVA***** KAP 52
*****KAP-IVB***** KAP 53
*****KAP-IVC***** KAP 54
*****KAP-IVD***** KAP 55
*****KAP-IVE***** KAP 56
*****KAP-IVF***** KAP 57
*****KAP-IVG***** KAP 58
*****KAP-IVH***** KAP 59
*****KAP-IVI***** KAP 60
*****KAP-IVJ***** KAP 61
*****KAP-IVK***** KAP 62
*****KAP-IVL***** KAP 63
*****KAP-IVM***** KAP 64
*****KAP-IVN***** KAP 65
*****KAP-IVP***** KAP 66
*****KAP-IVQ***** KAP 67
*****KAP-IVR***** KAP 68
*****KAP-IVS***** KAP 69
*****KAP-IVT***** KAP 70
*****KAP-IVU***** KAP 71
*****KAP-IVV***** KAP 72
*****KAP-IVW***** KAP 73
*****KAP-IVX***** KAP 74
*****KAP-IVY***** KAP 75
*****KAP-IVZ***** KAP 76
*****KAP-IVAA***** KAP 77
*****KAP-IVAB***** KAP 78
*****KAP-IVAC***** KAP 79
*****KAP-IVAD***** KAP 80
*****KAP-IVAE***** KAP 81
*****KAP-IVAF***** KAP 82
*****KAP-IVAG***** KAP 83
*****KAP-IVAH***** KAP 84
*****KAP-IVAI***** KAP 85
*****KAP-IVAJ***** KAP 86
*****KAP-IVAK***** KAP 87
*****KAP-IVAL***** KAP 88
*****KAP-IVAM***** KAP 89

INPERR = 0
TMFP=20.0
FOURPI = 4.0*3.14159236
FOURPI = FOURPI/3.0
DO 4 K = 1,151
DO 2 J = 1,50
2 FLUXS(K,J) = 0.0
4 FLUX(K) = 0.0
DO 6 I = 1,550
6 AI(I) = BLANK
DO 8 I = 1,1700
8 NI(I) = 0
DO 10 I=1,5660
10 FI(I) = 0.0
EPSLN = 1.0E-6
FUDGE = 1.0E-3
IOUT(1) = 1

C INSERT INPUT READ ROUTINE
C ADDRESSABLE INPUT ROUTINE
12 READ(5,100) NN,NL,NA,(AX(I),I=1,15)
1000 FORMAT(12,11,I9,15A4)
IF(NN.LE.0.OR.NN.GT.15)GO TO 16
IF(NA.LE.0.OR.NA.GT.550) GO TO 16
C ALPHANUMERIC READ
DO 14 I = 1,NN
J = I + NA - 1
14 AI(J) = AX(I)
IF(NL.LE.0) GO TO 12
GO TO 18
16 WRITE(6,1002) NN,NL,NA,(AX(I),I=1,15)
1002 FORMAT(36H0ERROR IN INPUT DATA CARD AS FOLLOWS //
1I3,I2,I10,15A5)
INPERR=1
IF(NL.GT.0) GO TO 18
GO TO 12

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```
C 18 WRITE(6,1016) (AI(I),I=1,45)
C FIXED
 20 READ(5,1004) NN,NL,NA,(NX(I),I=1,20)
1004 FORMAT(12,I1,I9,20I3)
  IF(NN.LE.0.OR.NN.GT.20) GO TO 24
  IF(NA.LE.0.OR.NN.GT.1700) GO TO 24
  DO 22 I = 1,NN
    J = I + NA - 1
22  NI(J) = NX(I)
  IF(IOUT(1).GT.0) WRITE(6,1006) NN,NL,NA,(NX(I),I=1,NN)
1006 FORMAT(13,I2,I10,20I4)
  IF(NL.LE.0) GO TO 20
  GO TO 26
24  WRITE(6,1008) NN,NL,NA,(NX(I),I=1,20)
1008 FORMAT(36H0ERRR IN INPUT DATA CARD AS FOLLOWS //
1I3,I2,I10,20I4)
  INPERR=1
  IF(NL.GT.0) GO TO 26
  GO TO 20
C FLOATING
 26 READ(5,1010) NN,NL,NA,(FX(I),I=1,5)
1010 FORMAT(12,I1,I9,5E12.5)
  IF(NN.LE.0.OR.NN.GT.5) GO TO 30
  IF(NA.LE.0.OR.NA.GT.5660) GO TO 30
  DO 28 I = 1,NN
    J = I + NA - 1
28  IF(J.LT.206C.OR.J.GE.2160) GO TO 28
  NBND=NEQBD(J-2059)
  IF(NBND.LT.2.OR.NBND.GT.3) GO TO 28
  FX(I)=FX(I)*FX(I)
28  FI(J) = FX(I)
  IF(IOUT(1).GT.0) WRITE(6,1012) NN,NL,NA,(FX(I),I=1,NN)
1012 FORMAT(13,I2,I10,1P5E13.5)
  IF(NL.LE.0) GO TO 26
  GO TO 32
30  WRITE(6,1014) NN,NL,NA,(FX(I),I=1,5)
1014 FORMAT(36H0ERRR IN INPUT DATA CARD AS FOLLOWS //
1I3,I2,I10,1P5E13.5)
1016 FORMAT(1H1,4X,15A4/(5X,15A4))
  INPERR = 1
  IF(NL.GT.0) GO TO 32
  GO TO 26
32  IF(INPERR.GT.0) CALL EXIT
C CALCULATE AMBIGUITY INDICES
  DO 48 I=1,NREG
    NBND = IABS(NBNDZN(I))
    DO 48 J=1,NBND
      L= IABS(LBD(J,I))
    NEQ = NEQBD(L)
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      WRITE(6,1018) J,RRC
1018 FORMAT(38H RECEIVER POINT COORDINATES FOR POINT 15,17H ARE INCORREKAP
1CT R=1PE15.5)
      GO TO 102
C CALCULATE THE CARTESIAN COORDINATES OF THE RECEIVER POINT
50 XRC = RRC*COS(PHIRC)
      YRC = RRC*SIN(PHIRC)
52 IERR3 = 0
      IF(IISCP.LE.0) GO TO 56
C CALCULATE THE LINEAR ABSORPTION COEFF. FOR THE SOURCE REGION
DO 54 L = 1,NRGY
      CORRL=0.0
      VOLSL=0.0
      SVOL(L)=0.0
      SVUC(L)=0.0
      XSECS(L) = C*0
      DO 54 I = 1,MAT
      54 XSECS(L) = XSECS(L)+XSECGL,I)*COMP(L,IISCP)
C BEGIN THE MAIN SOURCE POINT TO DETECTOR POINT LOOP
      NTOT=4*NGG+NGN+1
56 DO 80 L = 1,LSD
      NSOI = NSO(1)
      IF(IISTC.GT.1) NSOI=NSO(L)
      DO 80 N = 1,NSOI
      SINS = SIN(PHI(N,L))
      COSS = COS(PHI(N,L))
      XSOM = RSI(L)*COSS
      YSOM = RSI(L)*SINS
      DO 58 K = 1,2
      58 FSR(K) = FSIL,1,K)*FS(N,L+2,K)
      DO 80 M = 1,MSD
      IF(IISRC.LT.6) GO TO 60
      COSZ = COS(ZSI(M))
      XSO = XSOM*COSZ+SSOT(1)
      YSO = YSOM*COSZ+SSOT(2)
      ZSOM = SIN(ZSI(M))*RSI(L)+SSOT(3)
      GO TO 62
      60 XSO = XSOM+SSOT(1)
      YSO = YSOM+SSOT(2)
      ZSOM = ZSI(M)+SSOT(3)
      62 DO 64 K = 1,2
      64 FSP(K) = FSR(K)*FS(M,2,K)
      66 CALL LENGTH(SREG,SSO,XSO,YSO,ZSOM,YRC,ZRC,IERR1)
      IF(IERR1.LE.0) GO TO 68
      NERR = NERR + 1
```

```

1020 FORMAT(46HOTHE LENGTH ROUTINE HAS EXPERIENCED ERROR NO. 12)
      GO TO 104
68 CALL KERNEL(SREG,SSO, ANEUT,AGAM,UMFP, IERR2)
      IERR3 = MAX(IERR3,IERR2)
      A(2) = FSP(2)/(SSO*SSO)
      IF(INEUT(1).GT.0) FLUX(INTOT) = FLUX(NTOT) + A(2)*ANEUT(NGN+1)
      IF(INEUT(2).LE.0.AND.INEUT(3).LE.0) GO TO 72
      NST = 4*NGG
      DO 70 K = 1,NGN
      KN = NST+K
      70 FLUX(KN) = FLUX(KN) + A(2)*ANEUT(K)
      72 IF(IGAM.LE.0) GO TO 80
      74 A(1) = FSP(1)/(SSO*SSO)
      DO 78 K= 1,NGG
      IF(IISCP.LE.0) GO TO 76
      EPS = XSECS(K)*SSO
      IF(EPS.LT.SMFP) SVOL(K)=SVOL(K)+FSP(1)
      IF(EPS.LT.SMFP) GO TO 78
      76 FLUX(K) = FLUX(K) + A(1)*AGAM(K)
      KU = K+NGG
      FLUX(KU) = FLUX(KU) + A(1)*UAGAM(K)
      78 CONTINUE
      80 CONTINUE
      IF(IGAM.LE.0) GO TO 84
      DO 82 K = 1,NGG
      FLUX(K) = GSOUR(K)*FLUX(K)/FOURPI
      KG = K + NGG
      82 FLUX(KG) = GSOUR(K)*FLUX(KG)/FOURPI
      84 IF(INEUT(2).LE.0.AND.INEUT(3).LE.0) GO TO 88
      KS = 4*NGG
      DO 86 K = 1,NGN
      KN = KS + K
      86 FLUX(KN) = NSOUR(K)*FLUX(KN)/FOURPI
      88 IF(INEUT(1).GT.0) FLUX(INTOT) = AWNSOUR*FLUX(NTOT)/FOURPI
      IF(IISCP.LE.0) GO TO 92
      KS = 2*NGG
      DO 90 K = 1,NGG
      IF(SVOL(K).LE.0.0) GO TO 90
      VOLS(K) = FOUTPI*((SMFP/XSECS(K))**3)
      SVUC(K) = SVOL(K)*(1.-EXP(-SMFP))/XSECS(K)
      B1 = 3.*VOL(3,K,1)+BILD(2,K,1)
      B2 = 2.*B1
      CORR(K) = ((BILD(4,K,1)*SMFP+B2*BILD(2,K,1))*SMFP+BILD(2,KAP
      1K,1)+BILD(1,K,1))
      KN = KS + K
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FLUX(KN) = SVUC(K)
KN = KN + NGG
FLUX(KN) = SVUC(K)*CORR(K)
90 CONTINUE
92 IF(IISUM.LE.C) GO TO 96
DO 94 K=1,NTOT
FLUXS(K,J) = FLUXS(K,J) + FLUX(K)
JSUM = J + NDET
94 FLUXS(K,JSUM) =FLUXS(K,JSUM) + FLUX(K)
96 IF(IOUT(2).GT.0) GO TO 98
WRITE(6,1024) IZSO
1024 FORMAT(1H0,10X,37H CALCULATED RESULTS FOR SOURCE REGION 14 /)
JOUT=(J-1)*3+226
CALL FINSET(IFLUX,RRC,ZRC,PHIRC,JOUT,46)
98 DO 100 K=1,151
100 FLUX(K)=0.0
102 MON = 120.0
WRITE(6,1022) IZSO,J,NOVG,NOVN,TMFP,WON
1022 FORMAT(18H FOR SOURCE REGION 15,20H AND DETECTOR POINT 15,16H THERKAP
1E HAS BEEN 16.4H AND 16.25H PATH LENGTH CALCULATIONS/13H IN EXCESS KAP
2 OFF 6.1.32H MEAN FREE PATHS(GAMMA RAY) AND F6.1.34H GRAMS/CM**2(NEKAP
3 UTRON),RESPECTIVELY)
104 IF(IISUM.LE.1) GO TO 12
DO 108 J=1,NDET
108 JP=J
RRC =RCORD(1,JP)
ZRC =RCORD(2,JP)
PHIRC=RCORD(3,JP)
JOUT=(J-1)*3+226
WRITE(6,1026)
1026 FORMAT(1H0,10X,62H INTERMEDIATE SUMMARY RESULTS OVER A SUBSET OF S
OURCE REGIONS /)
CALL FINSET(IFLUX(1,J),RRC,ZRC,PHIRC,JOUT,76)
DO 106 K = 1,151
106 FLUXS(K,J) = 0.0
108 CONTINUE
110 IF(IISUM.LE.2) GO TO 12
110 JS=NDET+1
JF=2*NDET
DO 114 J=JS,JF
JP=J-JS+1
RRC =RCORD(1,JP)
ZRC =RCORD(2,JP)
PHIRC=RCORD(3,JP)
WRITE(6,1028)
1028 FORMAT(1H0,10X,63H CUMULATIVE SUMMARY RESULTS OVER ALL SUBSETS OF
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!SOURCE REGIONS /)
JOUT=(J-1)*3+226
CALL FINSET(FLUXS(1,J),RRC,ZRC,PHIRC,JOUT,1,6)
DO 112 K = 1,15
112 FLUXS(K,J) = 0.0
114 CONTINUE
GO TO 112
END
```

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$IBFTC FINSE NOLIST, DECK, NODD, M94/2, XRF7
      SUBROUTINE FINSET(FLUX, RRC, ZRC, PHIRC, JOUT, NP)
COMMON A1(550), NI(1700), FI(5660)
      EQUIVALENCE (FI(4750), RSPG), (FI(3560), ENG)
      EQUIVALENCE (FI(5050), RSPN), (FI(3462), ENN), (FI(5350), RSPA)
      EQUIVALENCE (NI(8), NRSPG), (NI(1), NGG)
      EQUIVALENCE (NI(9), NRSPN), (NI(2), NGN), (NI(10), NRSPA)
      EQUIVALENCE (AI(1), TITLE), (NI(21), INEUTT), (NI(24), ISCP)
      EQUIVALENCE (NI(20), IGM)
      DIMENSION RSPG(30,10), ENG(30)
      DIMENSION RSPN(30,10), ENN(30)
      DIMENSION RSPA(1,10)
      DIMENSION INEUT(3)
      DIMENSION FLUX(151)
      DIMENSION TITLE(550)
      IF(IIGAM.LE.0) GO TO 4
      NS = 1
      NF = NGG
      CALL FINOUT(FLUX, RSPG, ENG, TITLE(NP), TITLE(JOUT), TITLE(136),
1NRSPG, NGG, NS, NF, RRC, ZRC, PHIRC, 30)
      NS = NGG+1
      NF = NGG+NGG
      CALL FINOUT(FLUX, RSPG, ENG, TITLE(NP), TITLE(JOUT), TITLE(136),
1NRSPG, NGG, NS, NF, RRC, ZRC, PHIRC, 30)
      2 IF(ISCP.LE.0) GO TO 4
      NS = 2*NGG+
      NF = 3*NGG
      CALL FINOUT(FLUX, RSPG, ENG, TITLE(NP), TITLE(JOUT), TITLE(136),
1NRSPG, NGG, NS, NF, RRC, ZRC, PHIRC, 30)
      NS = 3*NGG+
      NF = 4*NGG
      CALL FINOUT(FLUX, RSPG, ENG, TITLE(NP), TITLE(JOUT), TITLE(136),
1NRSPG, NGG, NS, NF, RRC, ZRC, PHIRC, 30)
      4 IF(INEUT(2).LE.0 .AND. INEUT(3).LE.0) GO TO 6
      NS = 4*NGG+
      NF = 4*NGG+NGN
      CALL FINOUT(FLUX, RSPN, ENN, TITLE(NP), TITLE(JOUT), TITLE(166),
1NRSPN, NGN, NS, NF, RRC, ZRC, PHIRC, 30)
      6 IF(INEUT(1).LE.0) GO TO 8
      NS = 4*NGG+NGN+
      NF = NS
      CALL FINOUT(FLUX, RSPA, 1,0, TITLE(NP), TITLE(JOUT), TITLE(196),
1NRSPA, 1, NS, NF, RRC, ZRC, PHIRC, 1)
      8 RETURN
      END

```



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$IBFTC DEBNP NOLIST, DECK, NODD, M94/2, XRT
SUBROUTINE DFBN(A,NA,NS)
DIMENSION A(NA)
WRITE(5,100) NS
100 FORMAT(10X,15)
WRITE(5,102) (A(N),N=1,NA)
102 FORMAT(10X,EE12.5)
RETURN
END

$IBFTC DEBIP NOLIST, DECK, NODD, M94/2, XRT
SUBROUTINE DFRI(N,NI,NS)
DIMENSION N(NI)
WRITE(5,100) NS
100 FORMAT(10X,15)
WRITE(5,102) (N(I),I=1,NI)
102 FORMAT(10X,9I5)
RETURN
END

```



```
LEVEL1, SYSUT3,REW
$IBFTC KERN
      NOLIST, DECK, NODD, M94/2, XR7
      SUBROUTINE KERNEL(SMAT,SSO,ANEUT,AGAM,UAGAM,TMFP,IERR2)
      COMMON AI(550),NI(1700),FI(5660)
      EQUIVALENCE (NI( 1),NGG )*(NI( 2),NGN ),(NI( -3),MAT ) KER 1
      EQUIVALENCE (NI( 2),IGAM )*(NI( 2),INEUT ) KER 2
      EQUIVALENCE (NI( 35),KORD ),(NI( 40),IORD ),(NI( 400),NBILD ) KER 3
      EQUIVALENCE (NI( 350),XSECN )*(NI( 18),NOVN ) KER 4
      EQUIVALENCE (FI(3490),XSECN )*(FI(3550)*XSNREF),(FI(3550)*ALFA ) KER 5
      EQUIVALENCE (FI(4314),BKP )*(FI(4320),COM )*(FI(4620),CON ) KER 6
      EQUIVALENCE (FI(4720),XLAM )*(FI(3460),ENN ),(FI(3590),XSECG ) KER 7
      EQUIVALENCE (FI(4190),BILD )*(FI(4190),BILD ) KER 8
      EQUIVALENCE (NI( 3),KORD(5)),IORD(5),NBILD(5) KER 9
      EQUIVALENCE (NI( 5,4),XLAM(30),ENN(30),BILD(4,30,1) KER 10
      EQUIVALENCE (NI( 20,3),XSECN(30,20),ALFA(9)*BKP(4),COM(5,30,2) KER 11
      EQUIVALENCE (SMAT(100),ANEUT(31),AGAM(30),UAGAM(30) KER 12
      EQUIVALENCE (ANK(5)) KER 13
      *****KERNEL***** KER 14
      IF((INEUT(1).LE.0) GO TO 4
      C ALBERT-WELTON KERNEL
      J = NGN + 1
      SHYD = 0.0
      AN = 0.0
      DO 2 I = 1,MAT
      C TOTAL MEAN FREE PATHS - ALBERT-WELTON
      AN = AN + SMAT(I)*XSECN(I,1)
      C HYDROGEN MEAN FREE PATHS - ALBERT-WELTON
      2 SHYD = SHYD + SMAT(I)*XSECN(I,2)
      ANEUT(J) = ALFA(5)*EXP(-AN)
      IERR=1
      IF(SHYD.EQ.0.0) GO TO 4
      IF(SHYD.LT.ALFA(6)) ANEUT(J) = ANEUT(J)*EXP(-ALFA(7)*SHYD)
      IF(SHYD.GE.ALFA(6)) ANEUT(J) =(SHYD*ALFA(2)*(EXP(-ALFA(3)*(SHYD
      1 )*ALFA(4)))*ANEUT(J)*(ALFA(1)/
      ALFA(5)))
      2 IERR = 0
      4 IF((INEUT(2).LE.0.AND.INEUT(3).LE.0) GO TO 26
      AN = 0.0
      ANS = 0.0
      DO 6 I = 1,MAT
      6 AN = AN + SMAT(I)*XSECN(I,3)
      AN = AN / XSNREF
      IF(AN.LE.120.0) GO TO 8
      AN = AN - 120.0
      AN = 120.0
      NOVN = NOVN + 1
      KER 15
      KER 16
      KER 17
      KER 18
      KER 19
      KER 20
      KER 21
      KER 22
      KER 23
      KER 24
      KER 25
      KER 26
      KER 27
      KER 28
      KER 29
      KER 30
      KER 31
      KER 32
      KER 33
      KER 34
      KER 35
      KER 36
      KER 37
      KER 38
      KER 39
      KER 40
      KER 41
      KER 42
      KER 43
      KER 44
      KER 45
```

```

C MONOVARIANT MOMENTS   46
 8 IF(INEUT(2).LE.0) GO TO 14   KER 46
  IF(AN. GT. BKP(1)) J = 2   KER 47
  IF(AN. LE. BKP(1)) J = 1   KER 48
  DO 12 II = 1,NGN   KER 49
  ANEUT(II) = 0.0   KER 50
  DO 12 K = 1,5   KER 51
  ANEUT(II) = ANEUT(II)*AN + CON(K,II,J)   KER 52
  IF(ANEUT(II).GT.-88.0) GO TO 13   KER 53
  ANEUT(II)=0.0
  GO TO 12
 13 ANEUT(II) = EXP(ANEUT(II)-ANS*Xlam(II))   KER 54
 12 CONTINUE
 14 IF(INEUT(3).LE.0) GO TO 26   KER 55
C BIVARIANT MOMENTS
  DO 24 K = 1,NGN   KER 56
  IF(ENN(K).LE.BKP(1)) GO TO 16   KER 57
  WRITE(6,1000) K,ENN(K),BKP(1)   KER 58
  ENN(K) = BKP(1)   KER 59
 16 IF(ENN(K).GE.BKP(4)) GO TO 18   KER 60
  WRITE(6,1000) K,ENN(K),BKP(4)   KER 61
  ENN(K) = BKP(4)
 18 ANEUT(K) = r.0
  J=1
  IF(ENN(K).LT.BKP(2)) J=2   KER 62
  IF(ENN(K).LT.BKP(3)) J=3   KER 63
  KK = KORD(J)+1   KER 64
  II = IORD(J)+1   KER 65
  DO 20 L = 1,KK   KER 66
  ANK(L) = 0.0   KER 67
  DO 20 I = 1,II   KER 68
  IJ = II-I + 1   KER 69
 20 ANK(IJ) = ANK(L)*AN+CON(IJ,L,J)   KER 70
  DO 22 L = 1,KK   KER 71
  LJ = KK - L + 1   KER 72
 22 ANEUT(K) = ANEUT(K)*ENN(K) + ANK(LJ)   KER 73
  ANEUT(K) = ANEUT(K) - ANS*Xlam(K)   KER 74
  IF(ANEUT(K).GT.-88.0) GO TO 23   KER 75
  ANEUT(K)=0.0
  GO TO 24
 23 ANEUT(K) = EXP(ANEUT(K))   KER 76
 24 CONTINUE
 26 IF(IGAM.LE.r) GO TO 36   KER 77
  DO 34 K = 1,NGG   KER 78
  AGAM(K) = 0.0   KER 79
  AG = 0.0

```

```

DO 28 I = 1, MAT
28 AG = AG + SMAT(I)*XSEC(G(K,I))
IF(AG .LT. 88.0) GO TO 29
UAGAM(K)=.0.
GO TO 31
29 UAGAM(K) = EXP(-AG)
31 IF(AG.LE.TMFP) GO TO 30
C MORE THAN TMFP MEAN FREE PATHS
NOVG = NOVG + 1
AG = 2**I
30 DO 32 J = 1,4
JJ = 4 - J + 1
32 AGAM(K) = AGAM(K)*AG+3ILD(JJ,K,1)
34 AGAM(K) = AGAM(K)*UAGAM(K)
35 RETURN
1000 FORMAT(8RH' ERROR IN BREAKPOINTS FOR RANGE OF BIVARIANT POLYNOMIALS KER
1FOR MOMENTS DATA, ENERGY GROUP...14//,8H ENERGY=2PE15.5,14H ENERGY KER
2LIMIT=F15.E)
END

```

```

SIBFTC LENGTH NOLIST, DECK, NODD, M94/2, XRT
SUBROUTINE LENGTH(SMAT,SSO,XSO,YSO,ZSOM,XRC,YRC,ZRC,IERR)
COMMON AI(550),NI(1700),FI(5660)
EQUIVALENCE (NI( 3),MAT ),(NI( 25),NZSO ),(NI( 7),NREG )
LEN 1
LEN 2
LEN 3
LEN 4
EQUIVALENCE (NI( 200),NBNDZN),(NI( 100),NEQBD ),(NI(1100),NTRYZN) LEN 5
LEN 5
EQUIVALENCE (NI( 500),LBD ),(NI( 300),NCMPZN) LEN 6
LEN 6
EQUIVALENCE (FI(4313),FUDGE ),(FI(4312),EPSLN ),(FI(2460),COMP ) LEN 7
LEN 7
EQUIVALENCE (FI(1460),ABD ),(FI(1560),BBD ),(FI(1660),CBD ) LEN 8
LEN 8
EQUIVALENCE (FI(1760),XQBD ),(FI(1860),YQBD ),(FI(1960),ZQBD ) LEN 9
LEN 9
EQUIVALENCE (FI(2060),DBD ) LEN 10
LEN 10
DIMENSION NBNDZN(100),NEQBD(100),LBD(6,100),NTRYZN(6,100) LEN 11
LEN 11
DIMENSION NCMPZN(100) LEN 12
LEN 12
DIMENSION COMP(20,50),ABD(100),BBD(100),CBD(100),XQBD(100) LEN 13
LEN 13
DIMENSION YQBD(100),ZQBD(100),DBD(100) LEN 14
LEN 14
DIMENSION SMAT(100),R(6) LEN 15
LEN 15
2 CALL SLITET(4,K000FX)
GO TO(4,4),K000FX
4 CALL SLITET(1,K000FX)
GO TO(6,8),K000FX
6 CALL SLITE(1)
WRITE(6,1000)
1000 FORMAT(67HO ZONE BOUNDARY DISTANCE X Y
1          2)
8 DO 10 M=1,MAT
10 SMAT(M)=0.0
XO = XSO
Y=YSO
Z=ZSOM
AB = XRC -XO
BA = YRC-Y
GA = ZRC-Z
SRC=SQRT(AB*AB+BA*BA+GA*GA)
AB=AB/SRC
BA=BA/SRC
GA=GA/SRC
SSO=SRC
SMIN=FUDGE
NP=NZSO
12 XO=XO+AB*SMIN
Y=Y+BA*SMIN
Z=Z+GA*SMIN
SRC=SRC-SMIN
14 IL=NP
IU=NREG
16 DO 34 I=IL,IU
JU = IABS(NBNDZN(I))

```



```
DO 32 J=1,JU LEN 47
LL = LBD(J,I) LEN 48
L = TABS(LL) LEN 49
NEQ=NEQBD(LL) LEN 50
GO TO(18,20,22,24,26,28),NEQ LEN 51
18 R(J)=X0*(ABD(L)*X0+X0BD(L))+Y*(BBD(L)*Y+Y0BD(L)) LEN 52
1+Z*(CBD(L)*Z+Z0BD(L))-DBD(L) LEN 53
GO TO 30 LEN 54
20 R(J)=ABD(L)*(X0-X0BD(L))*2+BBD(L)*(Y-Y0BD(L))**2 LEN 55
1+ CBD(L)*(Z-Z0BD(L))**2-DBD(L) LEN 56
GO TO 30 LEN 57
22 R(J)=(X0-X0BD(L))**2+(Y-Y0BD(L))**2-DBD(L) LEN 58
GO TO 30 LEN 59
24 R(J)=X0-DBD(L) LEN 60
GO TO 30 LEN 61
26 R(J)=Y-DBD(L) LEN 62
GO TO 30 LEN 63
28 R(J)=Z-DBD(L) LEN 64
30 BDJ=FLOAT(LL) LEN 65
IF(SIGN(1.0,BDJ)*R(J)) 32,32,34 LEN 66
32 CONTINUE LEN 67
GO TO 42 LEN 68
34 CONTINUE LEN 69
IF(IU-NP)38,36,36 LEN 70
36 IL=I LEN 71
IU=NP-1 LEN 72
GO TO 16 LEN 73
38 IERR=1 LEN 74
CALL SLITET(1,K00CFX) LEN 75
GO TO(196,40),K00CFX LEN 76
49 CALL SLITE(1) LEN 77
GO TO 2 LEN 78
LEN 79
LEN 80
LEN 81
LEN 82
LEN 83
LEN 84
LEN 85
LEN 86
LEN 87
LEN 88
LEN 89
LEN 90
LEN 91
LEN 92
C 42 SMIN=SRC
KKK=0
IF(NBNDZN(1))96,96,44
44 DO 94 J=1,JU
LL = LBD(J,I)
L = TABS(LL)
NEQ=NEQBD(LL)
GO TO(46,48,52,54,56,58),NEQ
46 E=(ABD(L)*X0+X0BD(L)/2.0)*AB+(BBD(L)*Y+Y0BD(L)/2.0)*BA
1 +(CBD(L)*Z+Z0BD(L)/2.0)*GA
GO TO 50
48 E=ABD(L)*AB*(X0-X0BD(L))+BBD(L)*BA*(Y-Y0BD(L))
1 + CBD(L)*GA *(Z-Z0BD(L))
```

```

50 H=ABD(L)*AB    **2+BB0(L)*BA   **2+CBD(L)*GA   **2
50 GO TO 62
52 E=AB*(X0 - X0BD(L))+RA *(Y-Y0BD(L))
H=1.0-GA **2
GO TO 62
54 E=AB /2.0
GO TO 60
56 E=BA /2.0
GO TO 60
58 E=GA /2.0
60 H=0.0
62 IF(H)64,66,66
64 H=-H
E=-E
R(J)=-R(J)
LL = - LL
66 IF(H-EPSLN)68,68,78
68 IF(R(J)174,70,74
70 IERR=2
CALL SLITET(1,K000FX)
GO TO(106,72),K000FX
72 CALL SLITE(1)
GO TO 2
74 IF(ABS(E)-EPSLN)94,94,76
76 SBD=-R(J)/2.0/E
IF(SBD)94,94,90
78 IF(R(J))80,70,84
80 IF(H*SMIN**2+2.0*E*SMIN+R(J))94,94,82
82 QUAD=E**2-H*R(J)
GO TO 88
84 IF(E)86,94,94
86 QUAD=E**2-H*R(J)
IFI(QUAD-EPSLN)94,94,88
88 BDJI = FLOAT(1)
SBD=(-E+SIGN(SQRT(QUAD),BDJI))/H
90 IF(SBD-SMIN)92,94,94
92 CALL SLITE(4)
SMIN=SBD
KKK = LL
NP=NTRYZN(J,I)
94 CONTINUE
C
96 SMIN=SMIN+FUDGE
CALL SLITET(1,K000FX)
GO TO(98,100),K000FX
98 CALL SLITE(1)
LEN 94
LEN 95
LEN 96
LEN 97
LEN 98
LEN 99
LEN 100
LEN 101
LEN 102
LEN 103
LEN 104
LEN 105
LEN 106
LEN 107
LEN 108
LEN 109
LEN 110
LEN 111
LEN 112
LEN 113
LEN 114
LEN 115
LEN 116
LEN 117
LEN 118
LEN 119
LEN 120
LEN 121
LEN 122
LEN 123
LEN 124
LEN 125
LEN 126
LEN 127
LEN 128
LEN 129
LEN 130
LEN 131
LEN 132
LEN 133
LEN 134
LEN 135
LEN 136
LEN 137
LEN 138

```

```
139 LEN
140 LEN
141 LEN
142 LEN
143 LEN
144 LEN
145 LEN
146 LEN
147 LEN
148 LEN
149 LEN

      WRITE (6,102) I, KKK, SMIN, X0, Y, Z
102  FORMAT(17.4X,17.4X,1P4E12.5)
100  N=NCMPZN(I)
     DO 102 M=1,MAT
102  SMAT(M)=SMAT(M)+SMIN*COMP(M,N)
     CALL SLITF(4,K0)FX
     GO TO(12,104),K00)FX
104  IFRR=.1
105  CALL SLITF (.1)
     RETURN
     END
```



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```
1      SOURCE1, SYSUT3,REW
2      NOLIST, DECK, NODD, M94/2,XRT
3      SUBROUTINE SOURCE(A$0,R$1,Z$1,PHI$,FS ,INPERR )
4      COMMON AI(550),NI(1700),FI(5660)
5      EQUIVALENCE (NI( 27),ISRC ),(NI( 28),ISZC ) ,(NI( 29),ISTC )
6      EQUIVALENCE (NI( 50),LSO ),(NI( 51),MSO ) ,(NI( 52),NSO )
7      EQUIVALENCE (FI(  7),RS ),(FI( 28),ZS ) ,(FI( 49),PHI )
8      EQUIVALENCE (FI( 47),FSI ),(FI( 51),ASOI )
9      EQUIVALENCE (FI( 3),XI ),(FI( 5),ETA )
10     DIMENSION FSI(13,2),ASOI(2),ASO(2)
11     DIMENSION RSI(20),ZSI(20),PHI(20,20),FSI(21,22,2)
12     DIMENSION FSI(20,22,2),XI(2),ETA(2) ,NSO(20)
13     DIMENSION RSI(21),ZSI(21),PHI(21,20)
14     IF(ASOI(2).GT.0.0) GO TO 4
15     DO 2 K=1,21
16     DO 2 J=1,22
17     2   FSI(K,J,2) = FSI(K,J,1)
18     4   IF(ISRC.EQ.0) GO TO 78
19     DO 6 J = 1,3
20     DO 6 K = 1,2
21     6   FSI(J,K) = 0.0
22     DO 34 L = 1,LSO
23     IF(ISRC.GT.1) GO TO 8
24     C CYLINDRIC ,ISRC=1
25     C UNIFORM RADIAL SOURCE DISTRIBUTION
26     FS(L,1,1) = (RS(L+1)*RS(L+1)-RS(L)*RS(L))/2.0
27     FS(L,1,2) = FS(L,1,1)
28     GO TO 30
29     C COSINE ,ISRC=2
30     8   IF(ISRC.GT.2) GO TO 10
31     AS1=XI(1)*(RS(L+1)-XI(2))
32     AS2=XI(1)*(RS(L)-XI(2))
33     FS(L,1,1) = ((RS(L+1)*SIN(AS1)-RS(L)*SIN(AS2))/XI(1))
34     1   +(COS(AS1)-COS(AS2))/XI(1))/XI(1)
35     FS(L,1,2) = FS(L,1,1)
36     GO TO 30
37     10  IF(ISRC.GT.3) GO TO 14
38     C LINEAR ,ISRC=3
39     DO 12 K = 1,2
40     RS1=RS(L+1)
41     RS3=RS(L)
42     RS2=RS1*RS1
43     RS4=RS3*RS3
44     12  FS(L,1,K)=(FSI(L+1,1,K)-FSI(L,1,K))*(RS2+RS1*RS3+RS4)/3.0
45     -(FSI(L+1,1,K)*RS3-FSI(L,1,K)*RS1)*(RS1+RS3)/2.0
46     GO TO 30
```

```

14 IF(IISRC.GT.4) GO TO 18
C EXPONENTIAL WITH XI(1),S AS INPUT ,IISRC=4
DO 16 K = 1,2
AS1=EXP(XI(2)*(RS(L+1)-RS(L)))
AS2 = 1.0
16 FS(L,1,K) = ((AS1* (XI(2)*RS(L+1)-1.0)-XI(2)*RS(L)+1.0)*XI(1))/
1(XI(2)*XI(2))
GO TO 30
18 IF(IISRC.GT.5) GO TO 24
C EXPONENTIAL WITH XI(1),S CALCULATED FROM INPUT DISTRIBUTIONS, IISRC=5
DO 22 K=1,2
IF(FSI(L+1,1,K).EQ.FSI(L,1,K)) FS(L,1,K)= FSI(L,1,K)*(RS(L+1)*RS(L))
1+1)-RS(L)*RS(L)/2.0
IF(FSI(L+1,1,K).EQ.FSI(L,1,K)) GO TO 22
IF(FSI(L+1,1,K).GT.0.0.AND.FSI(L,1,K).GT.0.0) GO TO 20
WRITE(6,1002) K,L
INPERR=1
GO TO 22
20 XI(2)=( ALOG(FSI(L+1,1,K)/FSI(L,1,K)))/(RS(L+1)-RS(L))
XI(1)=FSI(L,1,K)
AS1=EXP(XI(2)*(RS(L+1)-RS(L)))
AS2 = 1.0
FS(L,1,K) = ((AS1* (XI(2)*RS(L+1)-1.0)-XI(2)*RS(L)+1.0)*XI(1))/
1(XI(2)*XI(2))
XI(1)=0.0
XI(2)=0.0
22 CONTINUE
GO TO 30
C SPHERICAL
24 IF(IISRC.GT.6) GO TO 26
C UNIFORM RADIAL DISTRIBUTION,IISRC=6
FS(L,1,1) = (RS(L+1)**3-RS(L)**3)/3.0
FS(L,1,2) = FS(L,1,1)
GO TO 30
C LINEAR, IISRC=7 OR GREATER
26 RS1 = RS(L+1)
RS2 = RS(L)
RS3 = RS1*RS1
RS4 = RS2*RS2
RS5 =(RS1+RS2)*(RS3+RS4)/4.0
RS6 =(RS3+RS1*RS2+RS4)/3.0
DO 28 K = 1,2
28 FS(L,1,K) = ((FSI(L+1,1,K)-FSI(L,1,K))*RS5-
1 (FSI(L+1,1,K)*RS2-FSI(L,1,K)*RS1)*RS6
30 DO 32 K = 1,2
32 FST(1,K) = FST(1,K) + FS(L,1,K)

```



```
34 RSI(L) = SQRT((RS(L+1)*RS(L)+RS(L)*RS(L+1))/2.0)
DO 58 M = 1,MSD
  IF(IISZC.GT.1) GO TO 36
C UNIFORM AXIAL DISTRIBUTION
  FS(M,2,1) = ZS(M+1)-ZS(M)
  FS(M,2,2) = FS(M,2,1)
  GO TO 54
36 IF(IISZC.GT.2) GO TO 38
  FS(M,2,1) = (SIN(ETA(1))*(ZS(M+1)-ETA(2))-SIN(ETA(1))*(ZS(M)-ETA(2))*
  1))/ETA(1)
  FS(M,2,2) = FS(M,2,1)
  GO TO 54
38 IF(IISZC.GT.3) GO TO 42
  DO 40 K=1,2
  40 FS(M,2,K)=((ZS(M+1)-ZS(M))*(FSI(M+1,2,K)+FSI(M,2,K))/2.0
  GO TO 54
42 IF(IISZC.GT.4) GO TO 46
  DO 44 K = 1,2
  44 AS1 = EXP(ETA(2)*(ZS(M+1)-ZS(M)))
  AS2 = 1.0
  FS(M,2,K) = (ETA(1)*(AS1-AS2))/ETA(2)
  44 CONTINUE
  GO TO 54
46 IF(IISZC.GT.5) GO TO 52
C EXPONENTIAL WITH ETA(1)*S CALCULATED FROM INPUT DISTRIBUTIONS
  DO 50 K=1,2
  50 IF(FSI(M+1,2,K).EQ.FSI(M,2,K))FS(M,2,K)=(ZS(M+1)-ZS(M))*FSI(M,2,K)
  IF(FSI(M+1,2,K).EQ.FSI(M,2,K)) GO TO 54
  IF(FSI(M+1,2,K).GT.0.0.AND.FSI(M,2,K).GT.0.0) GO TO 48
  WRITE(6,1004) K,M
  INPERR=1
  GO TO 50
  48 ETA(2) = (ALOG(FSI(M+1,2,K)/FSI(M,2,K)))/(ZS(M+1)-ZS(M))
  ETA(1) = FSI(M,2,K)
  AS1 = EXP(ETA(2)*(ZS(M+1)-ZS(M)))
  AS2 = 1.0
  FS(M,2,K) = (ETA(1)*(AS1-AS2))/ETA(2)
  ETA(1)=0.0
  ETA(2)=0.0
  50 CONTINUE
  GO TO 54
C SPHERICAL * POLAR DISTRIBUTION
  52 AS1 = SIN(ZS(M+1))
  AS2 = SIN(ZS(M))
  FS(M,2,1) = AS1-AS2
  SOU 93
  SOU 94
  SOU 95
  SOU 96
  SOU 97
  SOU 98
  SOU 99
  SOU 100
  SOU 101
  SOU 102
  SOU 103
  SOU 104
  SOU 105
  SOU 106
  SOU 107
  SOU 108
  SOU 109
  SOU 110
  SOU 111
  SOU 112
  SOU 113
  SOU 114
  SOU 115
  SOU 116
  SOU 117
  SOU 118
  SOU 119
  SOU 120
  SOU 121
  SOU 122
  SOU 123
  SOU 124
  SOU 125
  SOU 126
  SOU 127
  SOU 128
  SOU 129
  SOU 130
  SOU 131
  SOU 132
  SOU 133
  SOU 134
  SOU 135
  SOU 136
  SOU 137
  SOU 138
```

```

FS(M,2,2) = FS(M,2,1)          139
54 DO 56 K = 1,2
56 FST(2,K) = FST(2,K) + FS(M,2,K)
58 ZSI(M) = (ZS(M+1)+ZS(M))/2.0   SOU 140
IF(ISTC.GT.1) GO TO 60           SOU 141
NSTOP = NSO(1)+1                SOU 142
FST(3,1) = PHI(NSTOP,1) - PHI(1,1) SOU 143
FST(3,2) = FST(3,1)             SOU 144
GO TO 62                         SOU 145
60 NSTOP = NSO(L)+1              SOU 146
FST(3,1) = PHI(NSTOP,L) - PHI(1,L) SOU 147
FST(3,2) = FST(3,1)             SOU 148
62 DO 64 K = 1,2                SOU 149
ASOI(K) = ASOI(K)/(FST(1,K)*FST(2,K)*FST(3,K)) SOU 150
IF((ASOI(2).LE.0.0) ASO(2)=ASO(1)               SOU 151
IF((ASOI(2).LE.0.0) GO TO 66                 SOU 152
64 CONTINUE                      SOU 153
66 DO 76 L=1,LSO                SOU 154
IF(IISTC.LE.1) NSTOP = NSO(1)          SOU 155
IF(IISTC.GT.1) NSTOP = NSO(L)         SOU 156
NSTOP1= NSO(1)+1                  SOU 157
NSTOP2=NSTOP+1                   SOU 158
DO 74 N = 1,NSTOP               SOU 159
IF(IISTC.GT.1) GO TO 68           SOU 160
NSTOP1= NSO(1)+1                SOU 161
NSTOP2=NSTOP+1                   SOU 162
DO 74 N = 1,NSTOP               SOU 163
IF(IISTC.GT.1) GO TO 68           SOU 164
NSTOP1= NSO(1)+1                SOU 165
NSTOP2=NSTOP+1                   SOU 166
DO 74 N = 1,NSTOP               SOU 167
NSTOP1= NSO(1)+1                SOU 168
NSTOP2=NSTOP+1                   SOU 169
DO 74 N = 1,NSTOP               SOU 170
NSTOP1= NSO(1)+1                SOU 171
NSTOP2=NSTOP+1                   SOU 172
DO 74 N = 1,NSTOP               SOU 173
NSTOP1= NSO(1)+1                SOU 174
NSTOP2=NSTOP+1                   SOU 175
NSTOP1= NSO(1)+1                SOU 176
NSTOP2=NSTOP+1                   SOU 177
NSTOP1= NSO(1)+1                SOU 178
NSTOP2=NSTOP+1                   SOU 179
NSTOP1= NSO(1)+1                SOU 180
NSTOP2=NSTOP+1                   SOU 181
NSTOP1= NSO(1)+1                SOU 182
NSTOP2=NSTOP+1                   SOU 183
NSTOP1= NSO(1)+1                SOU 184

IF(IISTC.GT.2) GO TO 70
PHI(N,L) =(PHI(N+1,L)+PHI(N,L))*0.5
DPHI = PHI(N+1,L)-PHI(N,L)
GO TO 72
70 DT = NSTOP
DPHI=(PHI(NSTOP2,L)-PHI(1,L))/DT
DS = N
PHI(N,L) = PHI(1,L)+(DS-0.5)*DPHI
IF((PHI(NSTOP2,L).NE.PHI(NSTOP1,L)) WRITE(6,1000) L
72 FS(N,L+2,1) = ASO(1)*DPHI
74 FS(N,L+2,2) = ASO(2)*DPHI
76 CONTINUE
1000 FORMAT(53HERROR IN SOURCE DESCRIPTION FOR RADIAL INTERVAL....ISI)SOU
GO TO 86
78 DO 80 L = 1,LSO
RSI(L) = RS(L)
DO 80 K = 1,2
80 FS(L,1,K) = FSI(L,1,K)
DO 82 M = 1,MSO

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```

ZSI(M) = ZS(M)
DO 82 K = 1,2
 82 FSI(M,2,K) = FSI(M,2,K)
  NSOI=NSO(1)
  DO 84 N=1,NSOI
    PHI(N,1) = PHI(N,1)
  DO 84 K=1,2
    84 FSI(N,3,K) = FSI(N,3,K)
  1002 FORMAT(59HERROR IN RADIAL SOURCE DISTRIBUTION(1/GAMMA,2/NEUTRON),SOU
  1002 1000 I4,9HPOINT•••I4)
  1004 FORMAT(59HERROR IN AXIAL SOURCE DISTRIBUTION(1/GAMMA,2/NEUTRON),SOU
  1004 1000 I4,9HPOINT•••I4)
  86 RETURN
  END

```

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SIBFTC SIRR    NOLIST, DECK,NODD,M94/2,XR7      1
SUBROUTINE SIR(RS,ZS,RSIT,ZSIT,FSI,FSIT,LSO,MSO,LSIT,MSIT)   2
DIMENSION RS(21),ZS(21)                                SIR 3
DIMENSION RSIT(21),ZSIT(21),FSIT(21),FSI(21,22,2)      SIR 4
DO 8 K = 1,2                                         SIR 5
  LSOI = LSO + 1                                     SIR 6
  DO 4 L = 1,LSOI                                    SIR 7
    FSII(L,1,K) = SI(RSIT,FSIT(1,1,K),RS(L),LSIT,IND,1,1)  SIR 8
    MSOI = MSO + 1                                   SIR 9
    DO 6 M = 1,MSOI                                 SIR 10
      FSIM(2,M,K) = SI(ZSIT,FSIT(1,2,K),ZS(M),MSIT,IND,1,1) SIR 11
    6 CONTINUE                                         SIR 12
    RETURN                                            SIR 13
  8 CONTINUE                                         SIR 14
  RETURN                                            SIR 15
END                                                 SIR 16
SIBFTC PPLOTT NOLIST, DECK,NODD,M94/2,XR7
SUBROUTINE PPLOT
RETURN
END

SIBFTC LIBE    NOLIST, DECK,NODD,M94/2,XR7      1
SUBROUTINE LIBRE(ENG,ZAT,MAT,NGG,XSECGL)           2
COMMON AI(1550),NI(1700),FI(5660)                 3
DIMENSION GCO(32),GCT(532)                         4
DIMENSION ETABLE(28),GCI(28)                        5
DATA(ETABLE(1),I=1,28)/ 0.2,0.3,0.4,0.5,0.6,0.7,0.8,0.9,1.0,1.5,  LIB 6
12.0,2.5,2.8,3.0,3.5,4.0,4.5,5.0,5.5,6.0,6.5,7.0,7.5,8.0,8.5,9.0,  LIB 7
29.5,10.0 /                                         LIB 8
DATA(GCO(1),I=1,32) / 2.79370E-02,5.53742E-02,-8.83593E-03,9.79086LIB 9
1E-04,-4.48631E-04,-2.16137E-04,-1.69858E-04,-1.23547E-05,9.66869E-LIB 10
206,-5.75634E-06,5.1917E-06,5.27906E-07,-5.08811E-08,5.95621E-08,-1LIB 11
3.50369E-08,2.46673E-09,1.15458E-02,1.14923E-01,-4.93023E-02,-2.740LIB 12
486E-02,7.03833E-04,-5.98965E-03,9.06059E-03,-4.49765E-03,8.61480E-LIB 13
507,3.32807E-05,-2.97511E-05,-1.72444E-05,-2.30467E-08,-6.59005E-08LIB 14
6.8,41048E-08,9.63859E-08/                               LIB 15
DATA(GCT(1), I=1,28) / 0.321,.0329,.0341,.0355,.0371,.0388,.0406,LIB 16
2.0426,.0446,.0472,.0502,.0535,.0579,.0629,.0691,.0725,.0775,.0876,LIB 17
3.103,.0126,.0132,.0148,.0160,.0173,.0189,.0212,.0243/          LIB 18
DATA(GCT(1), I=29,56 1/,.0169,.0175,.0181,.0187,.0194,.0203,.0212,LIB 19
1.0221,.0231,.0244,.0257,.0276,.0295,.0322,.0349,.0367,.0395,.0442,LIB 20
2.0516,.0635,.0671,.0707,.0756,.0805,.0870,.0953,.0106,.0122 /          LIB 21
DATA(GCT(1), I=57,84 1/,.0154,.0159,.0162,.0168,.0173,.0180,.0185,LIB 22
2.0195,.0205,.0214,.0226,.0240,.0258,.0279,.0304,.0318,.0339,.0383,LIB 23
3.0448,.0550,.0579,.0614,.0655,.0697,.0752,.0827,.0922,.1063/          LIB 24
DATA(GCT(1), I=85,112) /,.0161,.0163,.0167,.0173,.0180,.0187,.0195,LIB 25
2.0203,.0211,.0222,.0234,.0247,.0266,.0286,.0313,.0327,.0349,.0394,LIB 26
3.0459,.0565,.0596,.0628,.0661,.0715,.0773,.0847,.0945,.109 /          LIB 27
DATA(GCT(1), I=113,140) /,.0182,.0183,.0187,.0193,.0216,LIB 28

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2.0234,• 0245,• 0258,• 0273,• 0292,• 0315,• 0346,• 036 • 0383,• 043 • LIB 29
3.0503,• 0623,• 0650,• 0683,• 0723,• 0775,• 0842,• 0929,• 104,• 118 / LIB 30
DATA(GCT(1),I=141,168) / • 0194,• 0195,• 0199,• 0205,• 0213,• 0219,• 0227,LIB 31
2.0236,• 0245,• 0256,• 0270,• 0284,• 0304,• 0327,• 0356,• 0372,• 0395,• 0444,LIB 32
3.0518,• 0636,• 0668,• 0707,• 0745,• 0805,• 0870,• 0953,• 106,• 122 / LIB 33
DATA(GCT(1),I=169,196) / • 02 • 0253,• 0206,• 0212,• 0218,• 0226,• 0233,LIB 34
DATA(GCT(1),I=225,252) / • 0203,• 0206,• 0210,• 0214,• 0218,• 0237,• 0398,• 0445,LIB 35
2.0242,• 0249,• 0261,• 0273,• 0289,• 0306,• 0331,• 0357,• 0374,• 0398,• 0445,LIB 35
3.0517,• 0636,• 0673,• 0707,• 0748,• 0805,• 0869,• 0955,• 106,• 123 / LIB 36
DATA(GCT(1),I=197,224) / • 0206,• 0208,• 0212,• 0216,• 0224,• 0229,• 0237,LIB 37
2.0245,• 0254,• 0264,• 0276,• 0289,• 0309,• 033 • 0359,• 0373,• 0396,• 0445,LIB 38
3.0518,• 0636,• 0666,• 0708,• 0743,• 0806,• 087 • 0953,• 107,• 123 / LIB 39
DATA(GCT(1),I=225,252) / • 0203,• 0206,• 0210,• 0214,• 0218,• 0224,• 0231,LIB 40
1.0238,• 0245,• 0255,• 0266,• 0281,• 0296,• 0319,• 0342,• 0358,• 0382,• 0422,LIB 41
2.0491,• 0662,• 0696,• 0736,• 0770,• 0716,• 0763,• 0826,• 0907,• 102,• 179 / LIB 42
DATA(GCT(1),I=253,280) / • 0218,• 0221,• 0225,• 0229,• 0233,• 0339,• 0446,LIB 43
1.0353,• 0260,• 0270,• 0281,• 0296,• 0312,• 0335,• 0359,• 0375,• 0401,• 0442,LIB 44
2.0514,• 0635,• 0665,• 0701,• 0749,• 0798,• 0865,• 0950,• 106,• 124 / LIB 45
DATA(GCT(1),I=281,308) / • 0215,• 0216,• 0220,• 0225,• 0229,• 0236,• 0242,LIB 46
2.0248,• 0254,• 0263,• 0274,• 0285,• 0303,• 0322,• 0348,• 0361,• 0383,• 0427,LIB 47
3.0496,• 0608,• 0641,• 0676,• 0716,• 0767,• 0833,• 0912,• 102,• 118 / LIB 48
DATA(GCT(1),I=309,336) / • 0228,• 0234,• 0238,• 0240,• 0242,• 0250,• 0255,LIB 49
2.0258,• 0266,• 0273,• 0286,• 0295,• 0315,• 0330,• 0360,• 0361,• 0385,• 0442,LIB 50
3.0512,• 0627,• 0640,• 0699,• 0725,• 0795,• 086,• 0944,• 106,• 122 / LIB 51
DATA(GCT(1),I=337,364) / • 0229,• 0230,• 0231,• 0235,• 0241,• 0242,• 0245,LIB 52
2.0255,• 0264,• 0271,• 0282,• 0300,• 0310,• 0335,• 0353,• 0375,• 0390,• 0432,LIB 53
3.0500,• 0614,• 0650,• 0683,• 0720,• 0777,• 0840,• 0922,• 103,• 120 / LIB 54
DATA(GCT(1),I=365,392) / • 0243,• 0245,• 0250,• 0252,• 0254,• 0260,• 0263,LIB 55
2.0272,• 0277,• 0285,• 0296,• 0308,• 0323,• 0342,• 0367,• 0375,• 0396,• 0447,LIB 56
3.0517,• 0635,• 0650,• 0706,• 0740,• 0802,• 0869,• 0954,• 107,• 125 / LIB 57
DATA(GCT(1),I=393,421) / • 0242,• 0245,• 0247,• 0248,• 0252,• 0255,• 0260,LIB 58
2.0264,• 0273,• 0279,• 0290,• 0299,• 0316,• 0330,• 0358,• 0368,• 0390,• 0436,LIB 59
3.0502,• 0617,• 0620,• 0685,• 0710,• 0780,• 0846,• 0928,• 104,• 122 / LIB 60
DATA(GCT(1),I=421,448) / • 0255,• 0256,• 0258,• 0260,• 0266,• 0270,• 0275,LIB 61
2.0278,• 0284,• 0292,• 0302,• 0314,• 0328,• 0348,• 0371,• 0383,• 0403,• 0448,LIB 62
3.0519,• 0635,• 066,• 0707,• 0742,• 0806,• 0874,• 0958,• 108,• 127 / LIB 63
DATA(GCT(1),I=449,476) / • 0252,• 0254,• 0257,• 0259,• 0262,• 0266,• 0271,LIB 64
1.0275,• 0280,• 0287,• 0295,• 0306,• 0318,• 0338,• 0358,• 0373,• 0395,• 0432,LIB 65
2.0498,• 0610,• 0646,• 0682,• 0730,• 0779,• 0845,• 0931,• 105,• 126 / LIB 66
DATA(GCT(1),I=477,504) / • 0243,• 0245,• 0247,• 0249,• 0251,• 0254,• 0258,LIB 67
1.0262,• 0266,• 0273,• 0280,• 0290,• 0301,• 0319,• 0338,• 0352,• 0372,• 0406,LIB 68
2.0468,• 0573,• 0606,• 0639,• 0685,• 0731,• 0790,• 0868,• 0978,• 117 / LIB 69
DATA(GCT(1),I=505,532) / • 0267,• 0269,• 0271,• 0273,• 0275,• 0279,• 0283,LIB 70
1.0287,• 0291,• 0298,• 0305,• 0316,• 0327,• 0346,• 0366,• 0380,• 0402,• 0439,LIB 71
2.0505,• 0620,• 0654,• 0689,• 0737,• 0786,• 0854,• 0938,• 106,• 126 / LIB 72
DO 12 N = 1,MAT LIB 73
NZ=(ZAT(N)-1.0)*28.0+1.0 LIB 74

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DO 12 K = 1,NGG
ENG1=ENG(K)
IF(ENG(K).GT.1.0)ENG1=1.0
IF(ENG(K).LT.2)ENG1=.2
2 IF(ZAT(N).GT.1.0) GO TO 6
DO 4 L=1,2
L1 = 28-L+1
NL=N7+L-1
4 SCI(LI)=GCT(NL)
XSECG(K,N)=SI(ETABLE,GCI,ENG1,28,IND,1,1)
GO TO 12
5 ENG1 = 1.0/FNG1
DO 8 I = 1,4
AG(I) = .0
DO 8 J = 1,4
JR=I-J+1+(I-1)*4
IF(ENG(K).GE.2.0) JR=JR+16
8 AG(I) = AG(I)*ENG1+GCN(JR)
ATOT = .0
DO 1 I = 1,4
13 I3 = 4-I+1
17 ATOT = ATOT*ZAT(N)+AG(I3)
XSECG(K,N) = ATOT
12 CONTINUE
14 RETURN
END

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2 6.54640E-02,3.27946E 00,-1.64393E-01,5.04304E-03,
3 -2.68716E-01,-4.74349E 00,7.45795E-01,-2.38346E-02,
4 2.84908E-01,3.39495E 00,-6.26720E-01,2.53658E-02,
5 -8.33177E-02,-8.37392E-01,1.66032E-01,-7.07268E-03,4*0.0/
C SET NO. 07 IRON DOSE 20MF,0.5 .LE.E.LE.10.0 BUI 44
DATA (CIJ(1),I=145,168)
1/1.01460E 00,-4.12104E-02,1.88074E-03,1.20198E-03,
2 -1.88657E-01,2.72752E 00,1.00217E-01,-9.83313E-03,
3 6.38649E-01,-3.76728E 00,-1.31988E-01,2.06002E-02,
4 -6.55159E-01,2.42384E 00,1.68976E-01,-1.75251E-02,
5 1.90742E-01,-5.54657E-01,-5.80710E-02,4.75673E-03,4*0.0/
C SET NO. 08 IRON ENERGY 20MF,0.5 .LE.E.LE.10.0 BUI 45
DATA (CIJ(1),I=169,192)
1/1.00790E 00,-6.14584E-02,1.67258E-02,-2.78863E-05,
2 -5.67036E-02,2.43299E 00,-6.93560E-02,1.48430E-03,
3 1.13800E-01,-3.33783E 00,3.27144E-01,-9.70973E-03,
4 -8.67732E-02,2.27697E 00,-2.53954E-01,1.03942E-02,
5 2.16742E-02,-5.47806E-01,5.89867E-02,-2.95818E-03,4*0.0/
C SET NO. 09 IRON ENERGY ABSORPTION 20MF,0.5 .LE.E.LE.10.0 BUI 46
DATA (CIJ(1),I=193,216)
1/1.00079E 00,-9.76981E-02,3.07480E-03,5.43036E-04,
2 -2.69391E-02,2.81567E 00,5.47069E-02,-3.05850E-03,
3 1.06797E-01,-3.14590E 00,-1.33763E-04,2.43681E-03,
4 -1.11725E-01,1.91566E 00,1.03003E-01,-1.57016E-03,
5 3.27028E-02,-4.17820E-01,-4.79248E-02,5.35111E-04,4*0.0/
C SET NO. 10 URANIUM DOSE 15MF,0.5 .LE.E.LE.10.0 BUI 47
DATA (CIJ(1),I=217,240)
1/1.01765E 00,4.24238E-02,-1.422252E-02,5.93162E-04,
2 -1.65482E-02,3.31303E-01,-2.90268E-03,-1.84501E-04,
3 5.84308E-03,-1.24381F-01,6.06091E-03,-7.15165E-05,
4 -8.56581E-04,1.62433E-02,-1.24162E-03,3.86282E-05,
5 4.63127E-05,-7.01931F-04,6.59820E-05,-2.68878E-06,4*0.0/
C SET NO. 11 URANIUM ENERGY 15MF,0.5 .LE.E.LE.10.0 BUI 48
DATA (CIJ(1),I=241,264)
1/1.01118E 00,7.8631E-02,-1.69975E-02,6.66422E-04,
2 -5.09776E-03,2.68738E-01,4.02443E-03,-4.01263E-04,
3 8.49397E-04,-1.13517E-01,2.85537E-03,4.66308E-05,
4 -9.01452E-05,1.34460E-02,-7.07914E-04,1.49376E-05,
5 3.14201E-06,-5.77921E-04,4.028867E-05,-1.38432E-06,4*0.0/
C SET NO. 12 URANIUM ENERGY ABSORPTION 15MF,0.5 .LE.E.LE.10.0 BUI 49
DATA (CIJ(1),I=265,288)
1/24*7.0/
C SET NO. 13 LFAD DOSE 15MF,0.5 .LE.E.LE.10.0 BUI 50
DATA (CIJ(1),I=289,312)
1/9.59342E-01,6.78254E-02,-2.26626E-02,6.39872E-04,
2 1.13722E-01,4.50412E-01,7.55191E-03,1.52094E-04, BUI 51



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3 -7.39816E-02,-2.15037E-01,5.10254E-03,-6.04837E-04,
4 1.87767E-02,4.05189E-02,-1.89332E-03,2.42263E-04,
5 -2.04254E-03,-3.40802E-03,1.93415E-04,-2.93865E-05,
6 7.93621E-05,1.06510E-04,-6.24306E-06,1.13914E-06/
C SET NO. 14 T IN ENERGY 15MFP,.5 .LE.E.LE.10.0 BUI 80
DATA (CIJ(1), I=313,336) BUI 81
1/1.02149E 0,5.09979E-02,-2.32211E-02,7.46627E-04, BUI 82
2 -1.82412E-02,4.38781E-01,2.69198E-02,-1.39819E-03, BUI 83
3 5.63188E-03,-2.16009E-01,-1.06625E-02,6.78031E-04, BUI 84
4 -7.34572E-04,4.19148E-02,2.32140E-03,-1.18583E-04, BUI 85
5 2.87951E-05,-3.63958E-03,-2.61850E-04,1.08867E-05, BUI 86
6 3.16311E-07,1.17723E-04,1.10200E-05,-4.19951E-07/ BUI 87
C SET NO. 15 LEAD ENERGY ABSORPTION 15MFP,.5 .LE.E.LE.4.0 BUI 88
DATA (CIJ(1), I=313,360) BUI 89
1/9.99305E-01,-3.49764E-01,2.89886E-02,1.28555E-03, BUI 90
2 1.13017E-02,2.79914E-02,-7.10002E-02,-4.75607E-03, BUI 91
3 -3.44367E-03,-2.20399E-01,-1.1108E-02,7.04843E-03, BUI 92
4 2.04092E-03,5.02301E-01,1.45444E-02,-2.31052E-03,8*0.0/ BUI 93
C LEAD ENERGY ABSORPTION 15MFP,4. .LE.E.LE.10.0 BUI 94
DATA (CIJ(1), I=361,384)
1/1.02626E 0,4.15814E-01,5.64179E-02,-4.83073E-03, BUI 95
2 -7.044724E-03,-6.14701E-02,-1.42153E-02,1.76866E-03, BUI 96
3 3.81090E-04,3.09491E-03,7.33689E-04,-1.07370E-04,12*0.0/ BUI 97
C SET NO. 17 T IN DOSE 15MFP,.5 .LE.E.LE.4.0 BUI 98
DATA (CIJ(1), I=385,468)
1/1.00210E 0,9.67305E-03,2.60452E-02,1.69043E-03, BUI 99
2 -5.96106E-03,1.72069E 0,-2.98161E-03,-6.86263E-03, BUI 100
3 7.46265E-03,-1.51375E 0,1.37709E-03,6.65061E-03, BUI 101
4 -2.26120E-03,3.95385E-01,-4.9948E-03,-1.76174E-03,8*0.0/ BUI 102
C T IN DOSE 15MFP,4.0 .LE.E.LE.10.0 BUI 103
DATA (CIJ(1), I=409,432)
1/1.01739E 0,5.94570E-01,6.71332E-02,-6.41932E-03, BUI 104
2 -5.18996E-03,-7.32374E-02,-1.27457E-02,2.20348E-03, BUI 105
3 2.96454E-04,2.88294E-03,6.04488E-04,-1.28823E-04,12*0.0/ BUI 106
C SET NO. 19 T IN ENERGY 15MFP,.5 .LE.E.LE.1.0 BUI 107
DATA (CIJ(1), I=433,456)
1/1.0128E 0,-2.22495E-02,1.57823E-02,1.52341E-03, BUI 108
2 -4.33121E-05,1.53249E 0,2.22903E-02,-6.24165E-03, BUI 109
3 -1.87779E-03,-1.22288E 0,0,-2.01839E-02,6.03325E-03, BUI 110
4 1.26439E-03,3.019313E-01,1.21119E-03,-1.58488E-03,8*0.0/ BUI 111
C T IN ENERGY 15MFP,4.0 .LE.E.LE.10.0 BUI 112
DATA (CIJ(1), I=457,480)
1/1.0283E 0,5.45757E-01,4.10166E-02,-3.39265E-03, BUI 113
2 -2.88528E-04,-7.96688E-02,-5.63538E-03,1.15437E-03, BUI 114
3 -1.03720E-04,3.93131E-03,8.91808E-05,-5.68006E-05,12*0.0/ BUI 115
C SET NO. 21 T IN ENERGY ABSORPTION 15MFP,.5 .LE.E.LE.4.0 BUI 116

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DATA (CIJ(I), I=481,594)
1/9. 89251E-01,-1.63257E-01,-4.60405E-02,5.33386E-03,
2 2.75399E-F2,2.37559E 00,3.09663E-01,-2.13112E-02,
3 -1.01933E-F2,-1.45840E 00,-3.44253E-01,2.27751E-02,
4 2.37931E-C3,3.13123E-01,9.59164E-02,-6.52903E-03,8*0.0/
          TIN      ENERGY ABSORPTION 15MFP,4.0 .LE.E.LE.10.0
C
C DATA (CIJ(I), I=505,528)
1/9. 81435E-03,7.86492E-01,1.21135E-02,1.08376E-03,
2 5.18941E-03,-1.42993E-01,1.83417E-03,-3.60388E-05,
3 -4.14965E-04,7.77610E-03,-3.65658E-04,1.39440E-05,12*0.0/
          SET NO. 23 TUNGSTEN DOSE   15MFP,.5 .LE.E.LE.4.0
C
C DATA (CIJ(I), I=529,552)
1/1. C0431E 00,-3.36453E-02,4.80434E-02,2.14764E-03,
2 -2.85210E-F2,1.61132E 00,-1.60382E-01,-6.82751E-03,
3 3.94170E-F2,-1.55661E 00,1.38446E-01,7.09074E-03,
4 -1.20482E-F2,4.13688E-01,-3.72797E-02,-2.03649E-03,8*0.0/
          TUNSTEN DOSE   15MFP,4.0 .LE.E.LE.10.0
C
C DATA (CIJ(I), I=553,576)
1/1. C2025E F0,4.36394E-01,7.25198E-02,-7.31585E-03,
2 -5.62425E-F3,-5.13127E-02,-1.69606E-02,2.67259E-03,
3 1.46397E-F4,2.58908E-03,7.10875E-04,-1.47831E-04,12*0.0/
          SET NO. 25 TUNGSTEN ENERGY   15MFP,.5 .LE.E.LE.4.0
C
C DATA (CIJ(I), I=577,600)
1/9. 84430E-F1,-1.85016E-02,2.39337E-02,2.49665E-03,
2 7.05675E-F2,1.26928E 00,-4.10157E-02,-9.93778E-03,
3 -7.45804E-F2,-1.10232E 00,1.85800E-03,1.11526E-02,
4 2.24076E-F2,2.69689E-01,4.22866E-03,-3.33528E-03,8*0.0/
          TUNGSTEN ENERGY   15MFP,4.0 .LE.E.LE.10.0
C
C DATA (CIJ(I), I=601,624)
1/1. C2106E 00,3.76582E-01,6.05907E-02,-5.26242E-03,
2 -6.54573E-C3,-4.62584E-02,-1.42730E-02,1.96063E-03,
3 2.65478E-F4,2.30558E-C3,6.39547E-04,-1.11335E-04,12*0.0/
          DATA (CIJ(I), I=625,648)
1/24*0.* /
          DATA (CIJ(I), I=649,672)
1/24*0.* /
          DATA (CIJ(I), I=673,696)
1/24*0.* /
          DATA (CIJ(I), I=697,720)
1/24*0.* /
          DO 6 K=1,120
          6 BILD(K,1,1) = 0.0
          NS=(NT1-1)*24+1
          NT = NT1
          DO 4 K = 1,NGG
          ENGI = ENGI(K)
          NSI = NS
          IF(ENGI.LT.FNGL(NT1))ENGI = ENGL(NT1)
          BUI 118
          BUI 119
          BUI 120
          BUI 121
          BUI 122
          BUI 123
          BUI 124
          BUI 125
          BUI 126
          BUI 127
          BUI 128
          BUI 129
          BUI 130
          BUI 131
          BUI 132
          BUI 133
          BUI 134
          BUI 135
          BUI 136
          BUI 137
          BUI 138
          BUI 139
          BUI 140
          BUI 141
          BUI 142
          BUI 143
          BUI 144
          BUI 145
          BUI 146
          BUI 147
          BUI 148
          BUI 149
          BUI 150
          BUI 151
          BUI 152
          BUI 153
          BUI 154
          BUI 155
          BUI 156
          BUI 157
        
```

```

IF(FNGI.GT.FNGH(NT1))FNGI = FNGH(NT1)
IF(NT1.GE.1.E-14.AND.FNGI.GT.4.E-14)NSI = NS + 24
IF(NT1.GE.1.E-15.AND.FNGI.GT.4.E-15)NT = NT+1
NN = NFNG(NT)
IF(NN.EQ.1)FNGI = 1.E-14/FNGI
DO 2 M = 1,4
DO 2 N = 1,6
NR = (K-N)*(M+N+NSI-1)
BILD(M,K,1) = BILD(M,K,1)*FNGI + CIJ(NR)
2 CONTINUE
4 TMFP = 2.E-14
IF(NT1.GT.9)TMFP = 1.E-14
RETURN
END

```

```

$IBFTC SIIR NDLIST. DECK, NODD, M94/2, XR7
FUNCTION SI(XTBL, YTBL, XX, NN, IND, INDE, INDF)
C
C      X=XX
C      N=NN
C      IND   = INDICATES TYPE-OF-EXTRAPOLATION THAT WAS USED (IF ANY)
C              (IND=1 INDICATES NO EXTRAPOLATION WAS NEEDED ON X)
C              (IND=2 INDICATES LOWER EXTRAPOLATION WAS NEEDED ON X)
C              (IND=3 INDICATES UPPER EXTRAPOLATION WAS NEEDED ON X)
C              (IND=4 INDICATES LOWER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=5 INDICATES TYPE-OF-UPPER-EXTRAPOLATION TO BE USED ON X)
C              (IND=6 INDICATES LOWER EXTRAP. ON X IS TO BE LINEAR)
C              (IND=7 INDICATES LOWER EXTRAP. ON X IS TO BE PARABOLIC)
C              (IND=8 INDICATES LOWER-LINEAR EXTRAP. ON X AND ERROR PRSII)
C              (IND=9 INDICATES LOWER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=10 INDICATES UPPER EXTRAP. ON X IS TO BE LINEAR)
C              (IND=11 INDICATES UPPER EXTRAP. ON X IS TO BE PARABOLIC)
C              (IND=12 INDICATES UPPER-LINEAR EXTRAP. ON X AND ERROR PRSII)
C              (IND=13 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=14 INDICATES UPPER-LINEAR EXTRAP. ON X AND ERROR PRSII)
C              (IND=15 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=16 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=17 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=18 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=19 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=20 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=21 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=22 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=23 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=24 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=25 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=26 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=27 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=28 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=29 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=30 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=31 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=32 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=33 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=34 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=35 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C              (IND=36 INDICATES UPPER-PARABOLIC EXTR. ON X AND ERROR PRSII)
C
C      2 IND=0
C      CHECK TO SEE IF LOWER OUT-OF-RANGE EXTRAPOLATION WILL BE NEEDED
C      IF(X-XTBL)4,6,8
C
C      LOWER OUT-OF-RANGE EXTRAPOLATION WAS FOUND NECESSARY (SET IND=1)
C      4 IND=1
C      6 II=2
C      GO TO 18
C
C      CHECK TO SEE IF UPPER OUT-OF-RANGE EXTRAPOLATION WILL BE NEEDED
C      8 IF(XTBL(N)-X)10,12,14
C
C      UPPER OUT-OF-RANGE EXTRAPOLATION WAS FOUND NECESSARY (SET IND=2)
C      10 IND=2
C      II=N
C      GO TO 18,12,18,12,14
C      12 II=N-1
C      GO TO 18
C

```

```

C   14  (X IS IN-RANGE   MAKE A POINT SEARCH ON X TO OBTAIN II)      SII  47
      NM1=N-1          SII  48
      DO 15  IK=2,NM1    SII  49
      II=IK            SII  50
      IF(XTBL(IK)-X)>16,18,18
      15 CONTNUF        SII  51
      SII  52
      SII  53
      SII  54
      SII  55
      SII  56
      SII  57
      SII  58
      SII  59
      SII  60
      SII  61
      SII  62
      SII  63
      SII  64
      SII  65
      SII  66
      SII  67
      SII  68
      SII  69
      SII  70
      SII  71
      SII  72
      SII  73
      SII  74
      SII  75
      SII  76
      SII  77
      SII  78
      SII  79
      SII  80
      SII  81
      SII  82
      SII  83
      SII  84
      SII  85
      SII  86
      SII  87
      SII  88
      SII  89
      SII  90
      SII  91
      SII  92
      SII  93
      SII  94
      SII  95

C   18  X1=XTBL(II-1)          MAKE A POINT SEARCH ON X TO OBTAIN II)      SII  47
      X2=XTBL(II)          SII  48
      Y1=YTBL(II-1)          SII  49
      Y2=YTBL(II)          SII  50
      C   CHECK IF (UPPER OR LOWER) EXTRAPULATION WAS FOUND TO BE NECESSARY
      IF(IND-1)26,23,22      SII  51
      C   LOWER EXTRAPULATION IS NEEDED - CHECK INDLE FOR TYPE TO BE USED
      23  GO TO (32,26,33,24),INDLE      SII  52
      C   UPPER EXTRAPULATION IS NEEDED - CHECK INDUE FOR TYPE TO BE USED
      22  GO TO (32,26,33,24),INDUE      SII  53
      C   ERROR PRINTOUT
      24  CALL ERROR(33H TABLE EXTRAPOLATED PARABOLICALLY)      SII  54
      C   26  X3=XTBL(II+1)
          Y2=YTBL(II+1)
          C   PARABOLIC INTERPOLATION OR EXTRAPOLATION
          28  SI=Y1+(1.0+(X2-X)/(X3-X1))*(Y2-Y1)*(X-X1)/(X2-X1)-(X3-X1)*(X-X1)/(X2-X1)-(X3-X1)*(X-X1)/(X2-X1)
              1X-X1)/(X3-X2)*(Y3-Y2)
              GO TO 34
          C   ERROR PRINTOUT
          30  CALL ERROR(23H TABLE EXTRAPOLATED LINEARLY)      SII  55
          C   LINEAR EXTRAPOLATION
          32  SI=Y1+(Y2-Y1)*(X-X1)/(X2-X1)
          34  RETURN
          END

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$ORIGIN LFVFL1, SYSUT3,REW
$IBFTC INOT NOLIST, DECK,NUOD,M94/2,XR7
      SUBROUTINE INOUT(ASO,RSI,ZSI,PHI1,FS,INPERR)
C INSERT COMMON, DIMENSION, AND EQUIVALENCE WHICH ARE IDENTICAL TO THE
C MAIN PROGRAM KAPV
      COMMON AI(550),NI(1700),FI(5660)
      EQUIVALENCE (AI( 1),TITLE )
      EQUIVALENCE (NI( 1),NGG ),(NI( 2),NGN )
      EQUIVALENCE (NI( 4),NCOMP ),(NI( 5),NDET )
      EQUIVALENCE (NI( 7),NREG ),(NI( 8),NRSPG )
      EQUIVALENCE (NI( 10),NRSPA ),(NI( 11),MATL )
      EQUIVALENCE (NI( 20),IGAM ),(NI( 21),INEUT )
      EQUIVALENCE (NI( 25),IZSO ),(NI( 26),ISORC )
      EQUIVALENCE (NI( 28),ISZC ),(NI( 29),ISTC )
      EQUIVALENCE (NI( 31),ISUM ),(NI( 32),IOUT )
      EQUIVALENCE (NI( 35),KORD ),(NI( 40),IORD )
      EQUIVALENCE (NI( 50),LSO ),(NI( 51),MSD )
      EQUIVALENCE (NI( 75),LSIT ),(NI( 76),MSIT )
      EQUIVALENCE (NI( 100),NEQBD ),(NI( 200),NBNDZN )
      EQUIVALENCE (NI( 400),NBILD ),(NI( 500),LBD )
      EQUIVALENCE (FI( 1),ASOI ),(FI( 3),XI )
      EQUIVALENCE (FI( 7),RS ),(FI( 28),ZS )
      EQUIVALENCE (FI( 470),FSI ),(FI(1400),GSOUR )
      EQUIVALENCE (FI(1460),ABD ),(FI(1560),BBD )
      EQUIVALENCE (FI(1760),XnBD ),(FI(1860),YOB0D )
      EQUIVALENCE (FI(2060),DBD ),(FI(2160),XYZ )
      EQUIVALENCE (FI(2460),COMP ),(FI(23460),ENN )
      EQUIVALENCE (FI(3490),XSECN ),(FI(3550),XSNREF )
      EQUIVALENCE (FI(3560),ENG ),(FI(3590),XSEC0 )
      EQUIVALENCE (FI(4310),SMFP ),(FI(4311),TMFP )
      EQUIVALENCE (FI(4312),EPSLN ),(FI(4313),FUDGE )
      EQUIVALENCE (FI(4320),COM ),(FI(4620),CON )
      EQUIVALENCE (FI(4750),RSPG ),(FI(5050),RSPN )
      EQUIVALENCE (FI(5360),RCORD ),(FI(5510),SSOT )
      EQUIVALENCE (FI(5533),RSIT ),(FI(5554),ZSIT )
      DIMENSION NSO(20),INEUT(3),IOUT(3),KORD(5)
      DIMENSION NBNDZN(100),NEQBD(100),NCMPZN(100),NBILD(100),LB0D(6,100)
      DIMENSION NTRYZN(6,100)
      DIMENSION ASO(12),XI(2),ETA(2),RS(21),ZS(21),PHI(21,20)
      DIMENSION FS(21,22,2),GSOUR(30),NSOUR(30)
      DIMENSION RSIT(21),ZSIT(21),FSIT(21,2,2)
      DIMENSION ASO(2),RSI(20),ZSI(20),PHI1(20,20),FS(20,22,2)
      DIMENSION ABD(100),BBD(100),CBD(100),DBD(100),XB0D(100),Y0BD(100)
      DIMENSION ZC0D(100),COMP(20,50),XYZ(3,100)
      DIMENSION ANEUT(31),AGAM(30),UGAM(30)
      DIMENSION ENN(30),XSECN(20,3),ALFA(9),ENG(30),XSECG(30,20)
      INO 1
      INO 2
      INO 3
      INO 4
      INO 5
      INO 6
      INO 7
      INO 8
      INO 9
      INO 10
      INO 11
      INO 12
      INO 13
      INO 14
      INO 15
      INO 16
      INO 17
      INO 18
      INO 19
      INO 20
      INO 21
      INO 22
      INO 23
      INO 24
      INO 25
      INO 26
      INO 27
      INO 28
      INO 29
      INO 30
      INO 31
      INO 32
      INO 33
      INO 34
      INO 35
      INO 36
      INO 37
      INO 38
      INO 39
      INO 40
      INO 41
      INO 42
      INO 43
      INO 44
      INO 45
      INO 46

```



```
DIMENSION BILD(4,30,1),BKP(4),COM(5,30,2),CON(5,5,4),XLAM(30)    INO 47
DIMENSION RSPG(30,10),RSPN(30,10),RCORD(3,50),RSPL(1,10)    INO 48
DIMENSION SSUT(3),TITLE(55),ZAT(20)    INO 49
INO 50
C INPUT DATA CONSISTENCY CHECK
IF(LIGAM.LE.1) GO TO 32
IF(NNGG.GT.0.OR.NGG.LE.30) GO TO 2
INPERR = 1
CALL POUT(56HND, OF GAMMA RAY GROUPS IN ERROR
1      )
2 IF(NRSPG.GT.0.OR.NRSPG.LE.10) GO TO 4
INPERR = 1
CALL POUT(56HND, OF GAMMA RAY RESPONSES IN ERROR
1      )
4 DO 6 J = 1,NRSPG
DO 6 K = 1,NGG
IF(RSPG(K,J).GT.0.0) GO TO 8
6 CONTINUE
INPERR = 1
CALL POUT(56HGMMA RAY RESPONSES ARE ALL ZERO
1      )
8 IF(ASOI(1).GT.0.0) GO TO 10
INPERR = 1
CALL POUT(56HGMMA RAY SOURCE MUST BE NON-ZERO AND POSITIVE
1      )
10 DO 12 K = 1,NGG
IF(GSDUR(K).GT.0.0) GO TO 14
12 CONTINUE
INPERR = 1
CALL POUT(56HGMMA RAY SOURCE SPECTRA MUST BE NON-ZERO AND POS.
1      )
14 DO 15 K = 1,NGG
IF(ENG(K).GT.0.0) GO TO 16
INPERR = 1
CALL POUT(56HALL GAMMA RAY ENERGIES MUST BE NON-ZERO AND POSITIVE
1      )
16 CONTINUE
DO 18 K = 1,NGG
DO 18 M = 1,MAT
IF(XSECG.GT.0.0) GO TO 20
18 CONTINUE
INPERR = 1
CALL POUT(56HGMMA RAY COEFF. MUST NOT ALL BE ZERO AND NON-POS.
1      )
20 DO 22 K = 1,NGG
DO 22 L=1,3
DO 22 J = 1,4
INO 51
INO 52
INO 53
INO 54
INO 55
INO 56
INO 57
INO 58
INO 59
INO 60
INO 61
INO 62
INO 63
INO 64
INO 65
INO 66
INO 67
INO 68
INO 69
INO 70
INO 71
INO 72
INO 73
INO 74
INO 75
INO 76
INO 77
INO 78
INO 79
INO 80
INO 81
INO 82
INO 83
INO 84
INO 85
INO 86
INO 87
INO 88
INO 89
INO 90
INO 91
INO 92
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IF(BILD(J,K,L).NE.0.0) GO TO 24
22 CONTINUE
INPERR = 1
CALL POUT(56HNO GAMMA RAY BUILDUP COEFFICIENTS ARE SPECIFIED
1)
24 DO 26 J = 1,NRSPG
DO 26 K = 1,NGG
IF(RSPG(K,J).GT.0.0) GO TO 28
26 CONTINUE
INPERR = 1
CALL POUT(56HNO GAMMA RAY RESPONSE FUNCTION DATA IS SPECIFIED
1)
28 DO 30 J = 1,2
DO 30 K = 1,LSO
IF(FSI(K,J).EQ.0.0) GO TO 30
INPERR = 1
CALL POUT(56HALL SOURCE DISTRIBUTION DATA MUST BE NON-ZERO AND POSIT
1.
30 CONTINUE
32 CONTINUE
DO 106 J=1,NREG
JU=IABS(NBNDZN(J))
DO 106 K=1,JU
IF(LBD(K,J).EQ.0.0) GO TO 102
IF(NTRYZN(K,J).EQ.0.0) GO TO 102
GO TO 106
102 INPERR = 1
CALL POUT(57H ALL GEOMETRY PARAMETERS(LBD AND NTRYZN) MUST BE NON-
1ZERO)
100 CONTINUE
WRITE(6,1000) NGG,NGN,MAT,NCOMP,NDET,NBOUND,NREG,NRSPG,NRSPN,NRSPAN
1,MATL
1000 FORMAT(22H1GENERAL PROBLEM INPUT
1 32H NO. OF GAMMA GROUPS.***** 14 /
2 32H NO. OF NEUTRON GROUPS.***** 14 /
3 32H NO. OF MATERIALS.***** 14 /
4 32H NO. OF COMPOSITIONS.***** 14 /
5 32H NO. OF DETECTORS.***** 14 /
6 32H NO. OF BOUNDARIES.***** 14 /
7 32H NO. OF REGIONS.***** 14 /
8 32H NO. OF RESPONSES(GAMMA).***** 14 /
9 32H NO. OF RESPONSES(NEUTRON).***** 14 /
1 32H NO. OF RESPONSES(ALBERT-WELTON) 14 /
1 32H NO. OF MATERIALS FROM LIBRARY. 14 /
1 WRITE(6,1002) IGAM,(LINEUT(I),I=1,3)
1002 FORMAT(20H CALCULATION OPTIONS
1 32H GAMMA RAY(C/NO,1/YES).***** 14 /
2 32H NEUTRON(C/NO,1/YES).***** 14 /

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131 INO 131
132 INO 132
133 INO 133

32H ALBERT-WELTON..... 14 /
32H MONOVARIANT MOMENTS..... 14 /
32H BIVARIANT MOMENTS..... 14 /
5 WRITE(6,1003) ISUM, IOUT(1), IOUT(2) 14 /

1003 FORMAT(14H*PRINT OPTIONS //)
1 32H SOURCE REGION SUMMATION 14 /
2 32H (O/NO, 1/SUM, 2/ST, 3/GT)*.... 14 /
3 32H INPUT PRINT(O/NO, 1/C1, 2/FULL). 14 /
4 32H OUTPUT PRINT 14 /
5 32H (1/ST AND GT ONLY, 2/ALL)*.... 14 /
6 WRITE(6,1004) ISCP, IZSO, ISORC, ISZC, ISTC
7 WRITE(6,1005) MSO, (ASO(I), I=1,LSO)

1004 FORMAT(27H*SOURCE CALCULATION OPTIONS //)
1 32H SOURCE ZONE COMPOSITION..... 14 /
2 32H MOST PROBABLE ZONE..... 14 /
3 32H SOURCE CALCULATION OPTION..... 14 /
4 32H RADIAL DISTRIBUTION..... 14 /
5 32H AXIAL OR POLAR DISTRIBUTION... 14 /
6 32H AZMUTHIAL DISTRIBUTION..... 14 /
7 32H SOURCE INTERVAL DATA // 14 /
8 32H NO. OF RADIAL..... 14 /
9 32H NO. OF AXIAL (OR POLAR)..... 14 /
10 32H NO. OF AZMUTHIAL..... 1014/(32X,10I4)
11 WRITE(6,1008) ASO(1), ASO(2), ASO(2), X(1), X(2), ETA(1), ETAIN(1),
12 ) IF (ISIT.GT.0) WRITF(6,1007) LSIT,MSIT
1007 FORMAT(26H*SOURCE INTERPOLATION DATA //)
1 32H NO. OF RADIAL VALUES..... 14 /
2 32H NO. OF AXIAL VALUES..... 14 /
1008 FORMAT(35H*SOURCE AND DISTRIBUTION PARAMETERS //)
1 35H GAMMA RAY SOURCE(INPUT)..... 1PE15.5/
2 18X 17H(NORMALIZED)..... E15.5/
3 35H NEUTRON SOURCE(INPUT)..... E15.5/
4 16X 19H(NORMALIZED)..... E15.5/
5 35H DISTRIBUTION PARAMETERS(X(1))... E15.5/
6 25X 10H(X(2))... E15.5/
7 25X 10H(ETA(1))... E15.5/
8 25X 10H(ETA(2))... E15.5/
9 WRITE(6,1010)
10 WRITE(6,1012)

1010 FORMAT(34H*SOURCE DISTRIBUTION DATA - RADIAL //)
1012 FORMAT(3X3HPT*,7X1CHCOORDINATE,14X9H GAMMA RAY,16X7H NEUTRON /
1 3X3HNO.,5X5HINPUT,7X8HMIDPOINT,2X,2(2X5HINPUT,7X10HNORMALIZINO
2FD) //)
30 34 I=1,LSO
34 WRITE(6,1014) I,RSI(1),FSI(1,1,1),FSI(1,1,2),FSI(1,1,2),FSI(1,1,2)
11,2)
1014 FORMAT(16,3X,1P7E12. 3)
LSOI = LSO+1

```

```

1016 WRITE(6,1016)(I,RS(I),FSI(I,1,1),FSI(I,1,2),I=LSOI,LSOI)
      FORMAT(16,3X,1PE12.3,12X,E12.3,E12.3)
      WRITE(6,1018)
      WRITE(6,1012)
1018 FORMAT(42H SOURCE DISTRIBUTION DATA - AXIAL OR POLAR //)
      DO 36 I=1,MSO
36   WRITE(6,1C14) 1,ZS(I),ZSI(I),FSI(I,2,1),FSI(I,2,2),FSI(I,1,INO
     12,2)
      MSOI = MSO + 1
      WRITE(6,1C16)(I,ZS(I),FSI(I,2,1),FSI(I,2,2),I=MSOI,MSOI)
      DO 40 J=1,LSO
40   WRITE(6,1020) J
      WRITE(6,1012)
1020 FORMAT(57H SOURCE DISTRIBUTION DATA - AZMUTHIAL FOR RADIAL INTERVAL
     1L 14 //)
      NSOI=NSOI(J)
      L=J+2
      DO 38 I=1,NSOI
38   WRITE(6,1014) I,PHI(I,J),PHII(I,J),FSI(I,L,1),FSI(I,L,1),FSI(I,L,2)
     1,FS(I,L,2)
      NSOI = NSOI+1
      WRITE(6,1016)(I,PHI(I,J),FSI(I,L,1),FSI(I,L,2),I=NSOI,NSOI)
     1F(1STC.LE.1) GO TO 42
42 CONTINUE
42   WRITE(6,1022)
      WRITE(6,1024)
1022  FORMAT(24H GAMMA RAY SPECTRAL DATA //)
1024  FORMAT(33H GROUP ENERGY SPECTRAL /,
     1          6H NO.,21X 4HDATA //)
     1   WRITE(6,1028)(K,ENG(K),GSOUR(K),K=1,NGG)
      WRITE(6,1026)
      WRITE(6,1024)
1026  FORMAT(22H NEUTRON SPECTRAL DATA //)
      WRITE(6,1N28)(K,ENN(K),NSOUR(K),K=1,NGN)
1028  FORMAT(16,3X,1P2E12.3)
      WRITE(6,1030)
1030 FORMAT(28H ZONE BOUNDARY SPECIFICATIONS //12H ZONE COMP.
     16(12H BND. ZONE //)
     1DO 44 K=1,NREG
      KS=1ABS(NBNDZ(K))
44   WRITE(6,1C32) K,NCMPZN(K),ILBD(KP,K),NTRYZN(KP,K),KP=1,KS)
     1FORMAT(14(1X,14.1X))
1032  WRITE(6,1034)
     1FORMAT(28H BOUNDARY EQUATION CONSTANTS//18H BND. NO. AND TYPE,2X
     12HAC,17X,2HXC,10X,2HBA,1CX,2HY0,1CX,2HCD,10X,2HDN//)
     1DO 46 K = 1,NBOUND
      INO 214
      INO 213
      INO 212
      INO 211
      INO 210
      INO 209
      INO 208
      INO 207
      INO 206
      INO 205
      INO 204
      INO 203
      INO 202
      INO 201
      INO 199
      INO 198
      INO 197
      INO 196
      INO 195
      INO 194
      INO 193
      INO 192
      INO 191
      INO 190
      INO 189
      INO 188
      INO 187
      INO 186
      INO 185
      INO 184
      INO 183
      INO 182
      INO 181
      INO 180
      INO 179
      INO 178
      INO 177
      INO 176
      INO 175
      INO 174
      INO 173
      INO 172
      INO 171
      INO 170
      INO 169
  
```



```
46 WRITE(6,1036) K, NEQBD(K), ABD(K), XQBD(K), BBD(K), YQBD(K), CRD(K), INO 215
1 ZBBD(K),DBD(K)
1036 FORMAT(2(16,3X),1P8E12.3)
DU 48 MS= 1,MAT,6
MF = MS+ 6 - 1
MF = MIN0(MF,MAT)
WRITE(6,1038) (MP,MP=MS,MF)
1038 FORMAT(25H,COMPOSITIONS BY MATERIAL //,
2 10H COMP.,30X12HMATERIAL NO. /
3 9H NO. 6(3X,16,3X) / )
DO 48 K = 1,NCOMP
48 WRITE(6,1040) K,(CUMP(M,K),M=MS,MF)
1040 FORMAT(16,3X,1P6E12.3)
WRITE(6,1042)
1042 FORMAT(27H,NEUTRON CROSS SECTION DATA /
1 44H MAT. / /,
2 44H NO. REMOVAL ETA / /)
DO 50 M = 1,MAT
50 WRITE(6,1014) M,(XSECN(M,K),K=1,3)
WRITE(6,1044)(ALFA(I,MP),MP=1,7),XSNREF
1044 FORMAT(34H,ALBERT-WELTON COEFF.,ALPHA(1)....1PE12.3/
1 21X,13H,ALPHA(2).... E12.3/
2 21X,13H,ALPHA(3).... E12.3/
3 21X,13H,ALPHA(4).... E12.3/
4 21X,13H,ALPHA(5).... E12.3/
5 21X,13H,ALPHA(6).... E12.3/
6 21X,13H,ALPHA(7).... E12.3//)
7 34H REFERENCE MATERIAL REMOVAL.....
DO 54 MS=1,MAT,6
MF=MS+6-1
MF=MIN0(MF,MAT)
WRITE(6,1046) (MP,MP=MS,MF)
1046 FORMAT(35H,O GAMMA RAY LINEAR ABSORPTION COEFF. //
1 40X,12HMATERIAL NO. //
2 18H COMP. ENERGY /
3 6H NO. 12X,6(3X,16,3X) / )
DO 52 K=1,NGG
52 WRITE(6,1014) K,ENG(K), (XSEC(G(K,M),M=MS,MF)
54 CONTINUE
WRITE(6,1048)
1048 FORMAT(25H,O GAMMA RAY BUILDUP COEFF. //
1 18H GROUP ENERGY 7X,2HB0,10X,2HB1,10X,2HB2,10X,2HB3/,INO 255
2 6H NO. )
DO 56 K=1,NGG
56 WRITE(6,1014) K,ENG(K),(BILD(J,K,1),J=1,4)
WRITE(6,1050)
```

```

1050 FORMAT(36HMONOVARIANT POLYNOMIAL MOMENTS DATA //           INO
1   20H GROUP ENERGY 4X,2HA0,10X,2HA1,10X,2HA2,10X,2HA3,INO 261
210X,2HA4/6H NO. ) INO 262
DO 58 J=1,2 INO 263
DO 58 K=1,NGN INO 264
58 WRITE(6,1014)K,ENN(K),(COM(I,K,J),I=1,5) INO 265
      WRITE(6,1052)
1052 FORMAT(36HBIVARIANT POLYNOMIAL MOMENTS DATA //           INO
1   32H GROUP ENERGY RANGE LIMITS 4X,2HC0,10X,2HC1,10X,2HINO 266
2C2,10X,2HC3,10X,2HC4/ INO 267
3   30H NO.      UPPER LOWER /) INO 268
DO 60 J=1,3 INO 269
J1=J+1 INO 270
KK=KORD(J)+1 INO 271
I1=IORD(J)+1 INO 272
DO 60 K=1,KK INO 273
60 WRITE(6,1014)J,BKP(J),BKP(J1), (CON(I,K,J),I=1,II) INO 274
      WRITE(6,1054)(K,ENN(K),XLAM(K),K=1,NGN) INO 275
1054 FORMAT(38H MOMENTS DATA EXTRAPOLATION PARAMETERS //           INO
1   30H GROUP ENERGY LAMBDA /) INO 276
2   6H NO. /(I6,3X,2E12.3)) INO 277
      WRITE(6,1056)
1056 FORMAT(24H GAMMA RAY RESPONSE DATA //) INO 278
DO 62 MS=1,NRSPG,5 INO 279
MF=MS+5-1 INO 280
MF=MIND(MF,NRSPG) INO 281
      WRITE(6,1058)(MP,MP=MS,MP) INO 282
MF=MIND(MF,NRSPG) INO 283
      WRITE(6,1058) INO 284
DO 62 K=1,NGG INO 285
62 WRITE(6,1040) K,ENG(K),(RSPG(K,MP),MP=MS,MP) INO 286
      WRITE(6,1040) ENERGY,14X, 17H RESPONSE FUNCTION / INO 287
1058 FORMAT(18H GROUP NO.,7X,I2,4(10X,12)) INO 288
2   6H
      WRITE(6,1060)
1060 FORMAT(22H NEUTRON RESPONSE DATA //) INO 289
DO 64 MS=1,NRSPN,5 INO 290
MF=MS+5-1 INO 291
MF=MIND(MF,NRSPN) INO 292
      WRITE(6,1058)(MP,MP=MS,MP) INO 293
DO 64 K=1,NGN INO 294
64 WRITE(6,1040)K,ENN(K),(RSPN(K,MP),MP=MS,MP) INO 295
      WRITE(6,1062)
1062 FORMAT(28H ALBERT/WELTON RESPONSE DATA //) INO 296
DO 66 MS=1,NRSPA,5 INO 297
MF=MS+5-1 INO 298
MF=MIND(MF,NRSPA) INO 299
      WRITE(6,1058)(MP,MP=MS,MP) INO 300
WON=1.0 INO 301
DO 66 MS=1,NRSPA,5 INO 302
MF=MS+5-1 INO 303
MF=MIND(MF,NRSPA) INO 304
      WRITE(6,1058)(MP,MP=MS,MP) INO 305
WON=1.0 INO 306

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```

DO 65 K=1,1
65 WRITE(5,104) K, MCN, IRSPLK(MP), MP=MS, MF)
      WRITE(5,104)
1064 FORMAT(27H RECEIVER POINT COORDINATES //
1       6H POINT ,16X,11HCOORDINATES /
2       6H ND). 8X,1HR, 11X,1HZ, 11X,1HO )
      WRITE(5,106)
1065 FORMAT(1H+,37X,1H-)
      DO 68 K=1,NDFT
68  WRITE(5,104) K, (RCORD(J,K), J=1,3)
      WRITE(6,1068)(SSOT(J), J=1,3)
1068 FORMAT(37H SOURCE POINT TRANSLATION COORDINATES //
1           SH Xooooo.1PE12.3 /
2           9H Yooooo. E12.3 /
3           9H Zooooo. E12.3 )
      WRITE(5,107)
1072 FORMAT(32H LIBRARY MATERIAL ATOMIC NUMBERS //
1           18H MAT.
2           19H ATOMIC /
2           NUMBER )
      DO 73 K=1,MATL
73  WRITE(5,104) K,ZAT(K)
      WRITE(5,1072) EPSLN,FUDGE,SMFP
1072 FORMAT(27H MISCELLANEOUS DATA //
1           29H BOUNDARY SEARCH EPSILON....1PE12.3 /
2           29H BOUNDARY SEARCH PARAMETER... E12.3 /
3           29H EMPIRICAL SOURCE SOLUTION /
4           29H MEAN FREE PATHS..... E12.3 / )
      RETURN
END

```

```
1      POT
2      POT
3      POT
4      POT
5      POT
6      POT
7      POT

$IBFTC PUTS NOLIST, DECK, NUDD, M94/2, XR7
SUBROUTINE PPUT(NAME)
DIMENSION NAME(14)
WRITE(6,100) (NAME(I), I=1,14)
100   FORMAT(10X,14A4)
      RETURN
      END
$ENTRY
$DATA
```